



U.S. Department of Energy
Office of Civilian Radioactive Waste Management

Waste Form Process Components

Presented to:

Nuclear Waste Technical Review Board

Presented by:

Patrick V. Brady

Waste Package Department

Bechtel SAIC Company, LLC

June 20-21, 2001

Las Vegas, NV

YUCCA
MOUNTAIN
PROJECT

Outline

- **Overview**
- **Unquantified uncertainties**
 - **In-Package chemistry**
 - **Np and Pu dissolved concentrations**
 - **In-package transport**
 - **Cladding**
- **Low temperature implications**
- **Other lines of evidence**

Overview

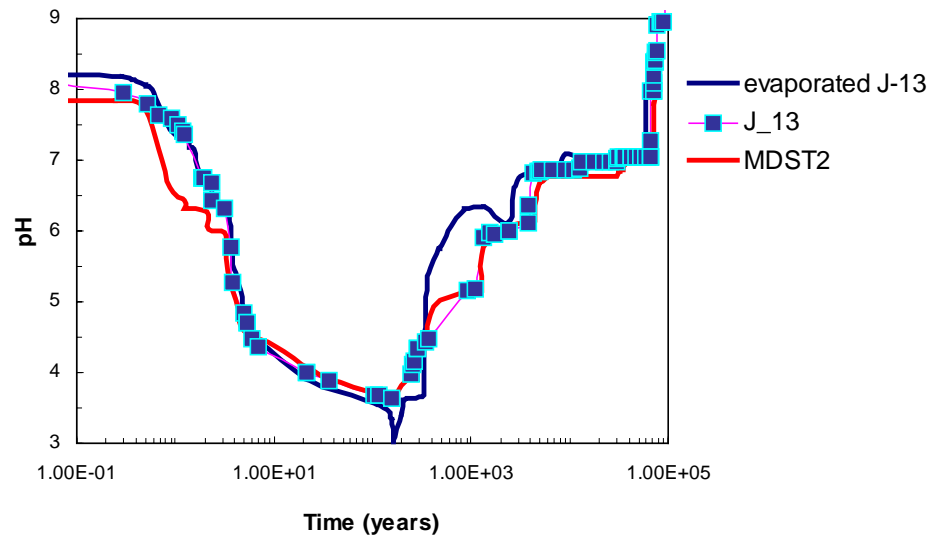
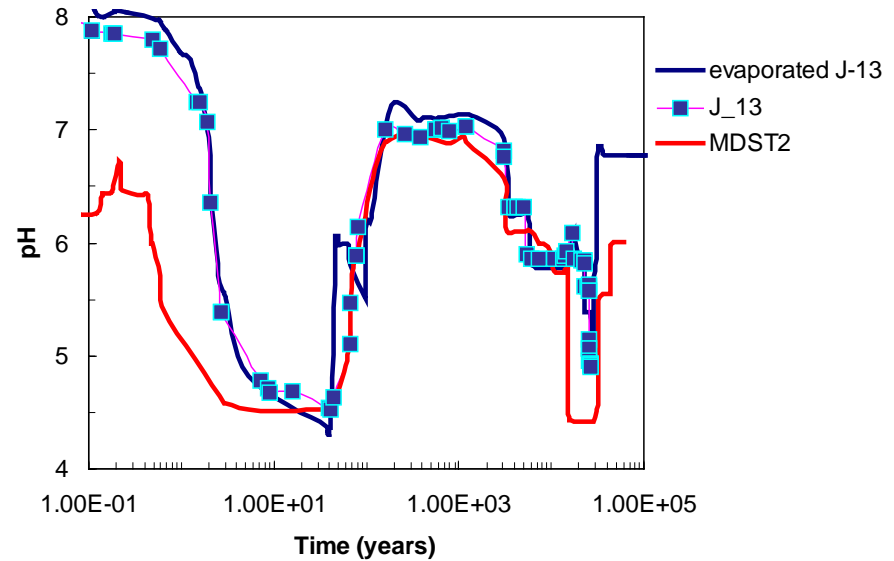
- **EQ3/6 uses steel, glass, and fuel degradation rates to predict major element behavior (pH, Cl-, F-) inside the waste form as a function of time**
- **Solubility response surfaces for Np, Pu, U, Tc are calculated and mapped onto abstracted in-package reaction paths to predict dissolved concentrations.**
- **Degradation product masses and sorption of radionuclides onto degradation products is estimated**
- **Diffusive transport from breached no-seepage CSNF WP's is estimated (instantaneous advective release assumed for all others)**
- **Clad integrity is predicted as a function of time**

In-Package Chemistry

Non J-13 inputs

Steel degradation

Glass degradation



Conclusion: Input fluid made so little difference in pH trajectories that no change needed.

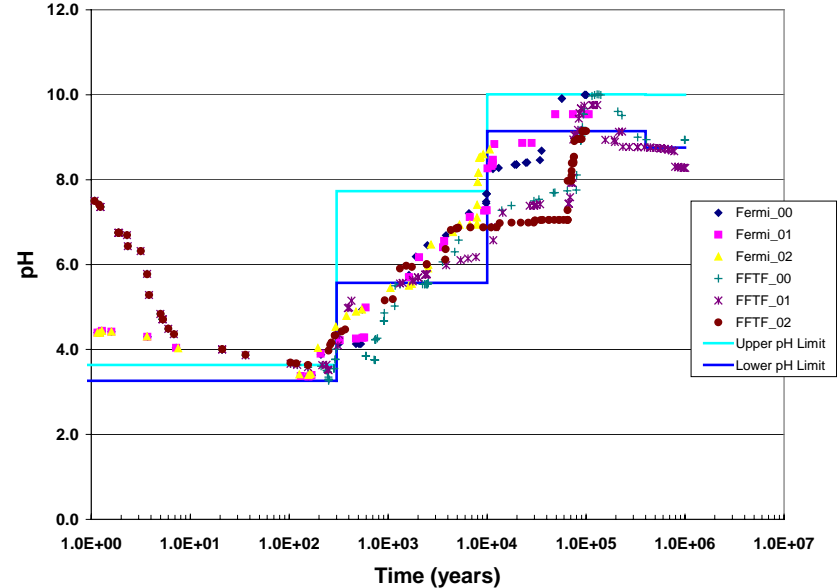
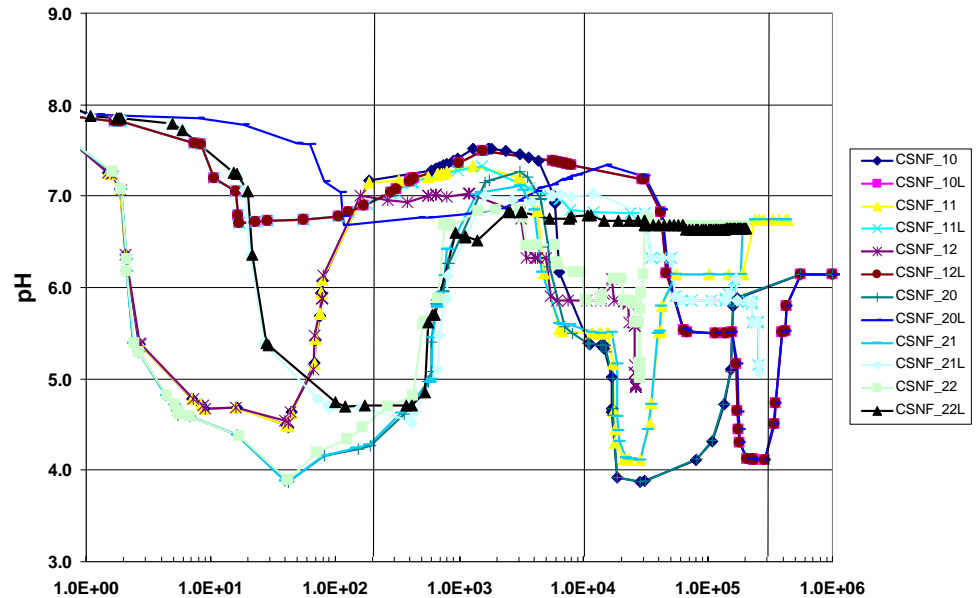
Supplemental In-Package Chemistry

Non J-13 inputs

Steel degradation

Glass degradation

Conclusion: pH trajectories resolved into more time steps; lower flow rates considered.



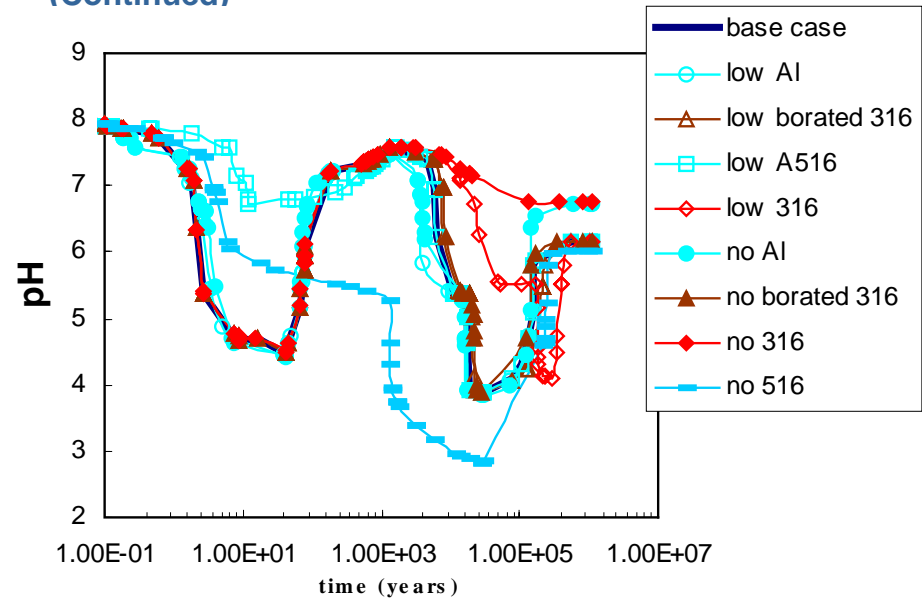
Supplemental In-Package Chemistry

(Continued)

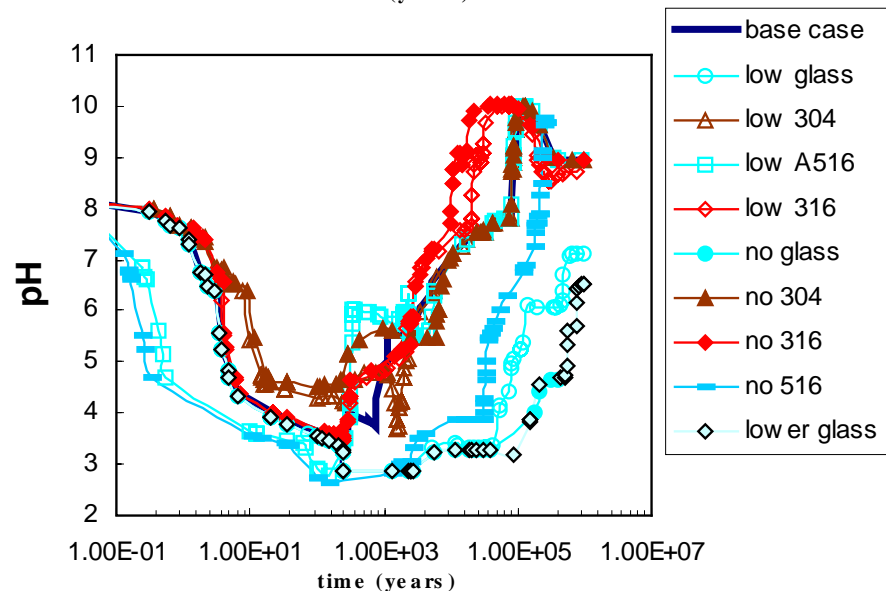
Non J-13 inputs

Steel degradation

Glass degradation

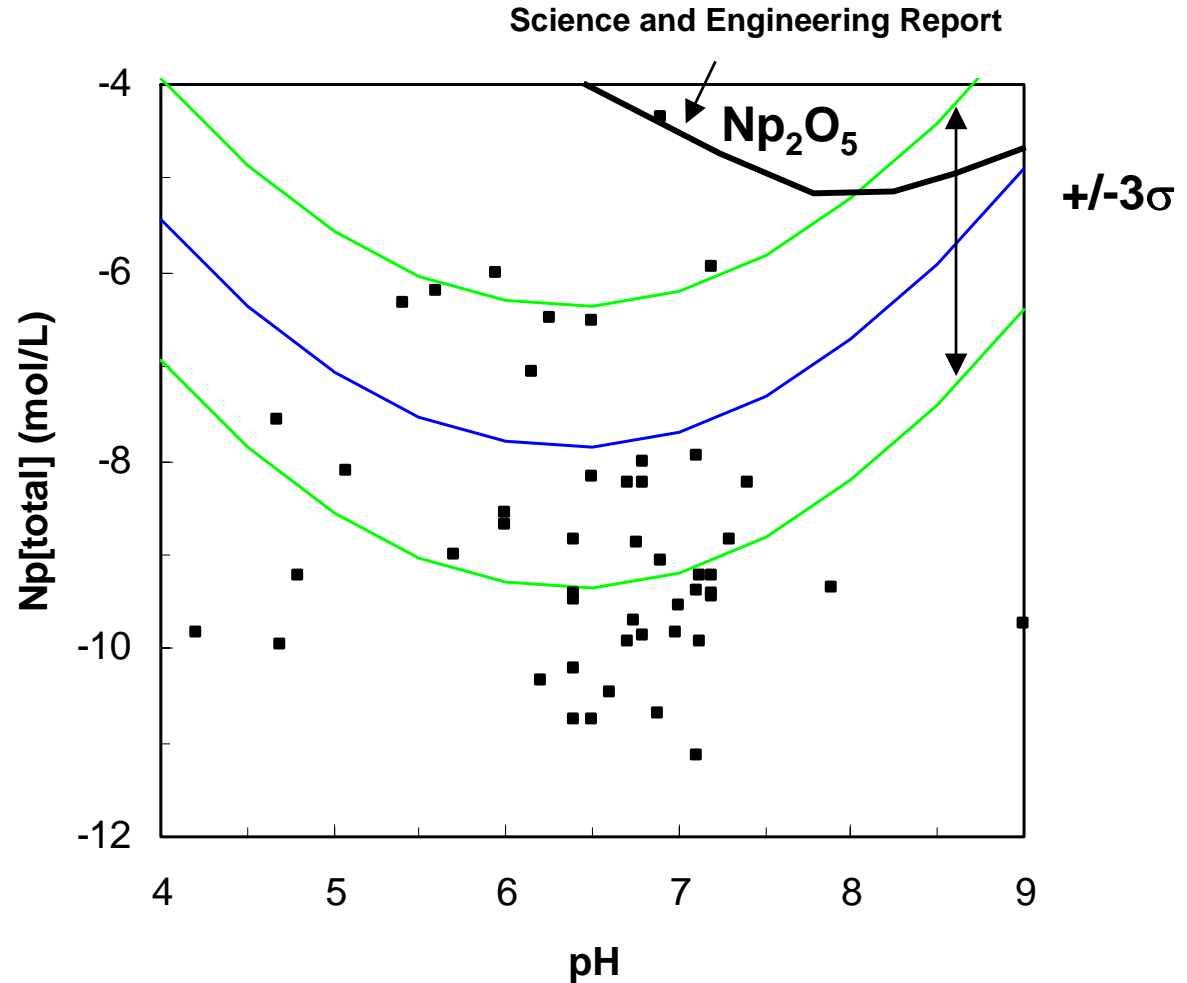


Conclusion: Glass-free and A516-free cases most significant.



Np Dissolved Concentrations

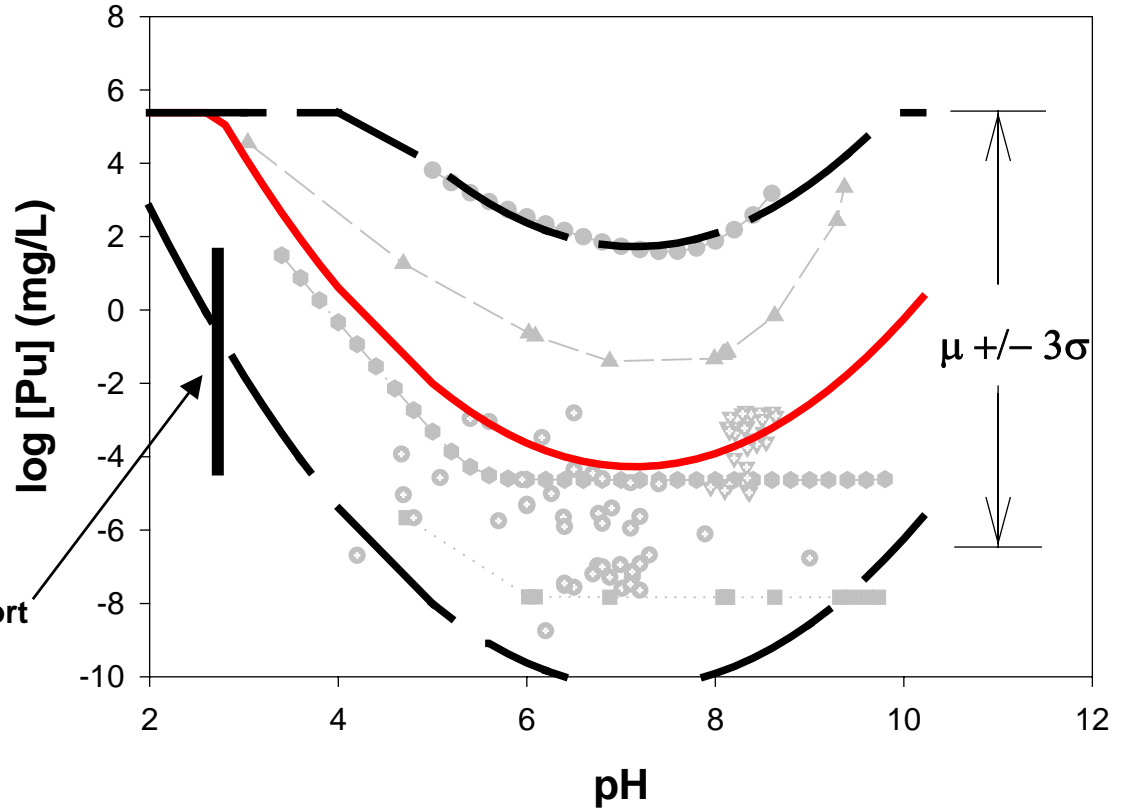
Conclusion: Secondary phase model results consistent with ANL drip tests, and indicate lower mean, more realistic uncertainty.



Pu Dissolved Concentrations

Conclusion: Newer model results in lower mean, more mechanistic basis.

Science and Engineering Report



In-Package Transport

- Estimate amount of H₂O sorbed (Fe₂O₃ and ZrO₂) and water saturation in package
- Estimate diffusion coefficients (Archie's Law) and cross-sectional area of H₂O films
- Calculate diffusion through corrosion products and breaches and along fuel rods

In-Package Transport

(Continued)

In-Package Sorption

- Calculate masses of sorbing phases (iron and copper oxides)
- Establish range of sorption K_d 's

Cladding

- **Initial cladding perforation**
- **Creep and stress corrosion cracking perforation (triangular-1.05:2.44:19.4% vs. uniform-0.0:0.5%)**
- **Local corrosion (F- and Ferric chloride)**
- **Seismic failure (1.1E-6/yr vs. CCDF-4.9E-6:2.7E-12)**
- **Rock overburden failure**
- **Clad unzipping (triangular-1:40:240 vs. CCDF-1:15,000)**

Low Temperature Implications

In-package chemistry and Dissolved concentrations

- Retrograde solubilities
- Lower gas solubilities at higher T
- Corrosion rates

In-package diffusion

- Effective diffusion coefficient
- Water adsorption
- Evaporation

In-package sorption

- Increases with temperature

Cladding

- T less important below 350°C
- Weak WF dissolution rate included in clad unzipping rate

Other Lines of Evidence

In-package chemistry and Dissolved concentrations

- Glass degradation – field studies
- Steel degradation – persistence of reduced iron at YM and elsewhere
- Thermodynamic data – experimental measurements
- Np and Pu concentrations – ANL and PNL drip and batch tests

In-package diffusion

In-package sorption

- Measurements made at hazardous waste sites, sequential extractions of bomb pulse radionuclides

Cladding

- Extensive experimental literature