

**U.S. Department of Energy** 



### Mass and Activity of Key Radionuclides Potentially Released from the Unsaturated Zone Over Time

Presented to: Nuclear Waste Technical Review Board

Presented by: Bruce A. Robinson Los Alamos National Laboratory

February 01, 2006 Las Vegas, Nevada

# Outline

- Unsaturated Zone (UZ) Radionuclide Transport Conceptual Models and Technical Basis
- Implementation of Unsaturated Zone Radionuclide Transport model for Total System Performance Assessment (TSPA)
- Results
  - Representative case
  - Sensitivity to flow model parameters
  - Diffusion processes and parameters
  - Fracture versus matrix releases
  - Spatial variability

### Conclusions





### Unsaturated Zone Transport Conceptual Model



- Combined fracture and matrix flow: dual permeability model formulation
- Radionuclide transport through fractures and matrix: advection, diffusion, sorption, colloid-facilitated transport
- Radionuclide transport is simulated using ambient flow fields
- Releases to either the fractures or matrix





**Department of Energy •Office of Civilian Radioactive Waste Management** YMRobinson\_NWTRB\_020106.ppt

abq0063G100 a

### **Scientific Basis**

- Fracture vs. matrix flow
  - Busted Butte experiment results confirm matrix flow in vitric Calico Hills units
  - Alcove 8, Niche 3 results confirm the process of fracture flow and matrix diffusion in the TSw units
  - CI-36 results suggest the possibility of fracture-dominated transport of conservative species through the unsaturated zone
  - Model of combined fracture and matrix flow and transport is consistent with many observations of solute transport in vadose zones (beyond Yucca Mountain)
- Flow and Transport Parameters
  - Process flow and transport models are informed by data sets either by direct calibration (e.g. water content, matric potential) or by consistency checks (CI-36, C-14)





### Implementation of UZ Radionuclide Transport in TSPA Model

- Dual permeability particle tracking model, with probabilistic travel time delays to account for sorption and diffusion
- Full decay-chain capability
- Particle release locations and mass per particle are determined dynamically from engineered barrier system radionuclide mobilization and transport calculations
- Particles are released into the fracture or matrix continuum
- Radionuclide mass versus time at various locations at the water table is computed and input to the saturated zone model
- Validation achieved by comparison to 1, 2, and 3D models, including the UZ transport process model





### **Radionuclides Considered**

- Conservative: C-14, I-129, Tc-99
- Weakly Sorbing: Np-237, U-232, U-234, U-235, U-236, U-238
- Strongly Sorbing: Am-241, Am-243, Cs-135, Cs-137, Pa-231, Pu-236, Pu-239, Pu-240, Pu-242, Ra-226, Sr-90, Th-229, Th-230, Th-232
- Colloid-Facilitated Transport: most strongly sorbing radionuclides





# **Advective Transport**

- 3D, steady state, dualpermeability flow fields from UZ flow model
- Instantaneous transition of flow field from one climate state to another
- Water table rise for future, wetter climates
- Uncertainty from infiltration model is propagated through the UZ transport model
- Sensitivity to flow-model parameters is explored through sensitivity analyses

### **Climate-related variability**





## **Transport Parameters and Uncertainties**

- The unsaturated zone transport model incorporates probabilistically defined parameters to propagate uncertainty through the UZ model
- Sorption reversible, equilibrium sorption model
  - Distributions developed for each radionuclide
  - Sorption to colloids is included
- Diffusion
  - Diffusion coefficient distribution from laboratory measurements
  - Uncertainty distributions for geometric parameters (aperture, fracture spacing)
  - Conceptual-model uncertainty for fracture-matrix interactions
- Colloid Transport Properties





# Mathematical Implementation Using Particle Tracking – Colloid Transport Model

- Reversible Sorption Type Colloid
  - Colloid partitioning coefficient (K<sub>c</sub>) describes the relative amount of radionuclide on colloids versus that in the aqueous phase
  - Only aqueous phase radionuclides can sorb or diffuse into the rock matrix
- Irreversible Sorption Type Colloid
  - Advective transport without diffusion into the rock matrix
  - Size exclusion model to prevent transport from fractures into some matrix units
  - Retardation via reversible filtration within the fracture continuum
  - A small fraction of the colloid inventory transports without retardation due to filtration (the "fast fraction")



- Radionuclide
- Sorbed Radionuclide



Reversible Sorption Type Colloid shown with radionuclide temporarily attached

Reversible Sorption Type Colloid shown without radionuclide attached

Irreversible Sorption Type Colloid shown with radionuclide permanently attached abq0063G031.ai



## **Results – Normalized Breakthrough Curves**

- This breakthrough curve (BTC) method measures the model-predicted distribution of arrival times at the water table
  - Representative case and other sensitivity analyses assume a release over the entire repository footprint
  - Method particles are introduced at time 0, BTC is the cumulative number that arrive at the water table at various times
  - Curves are normalized to the number of particles introduced, so should approach 1 at long times in the absence of radioactive decay
  - Radionuclides participating in decay chains are introduced both as the radionuclide itself and as a parent which decays to the radionuclide – because all species are introduced in the simulation in this way, a few radionuclide BTCs go above 1





# Results – Representative Case, Various Radionuclides

**Glacial-transition climate, mean infiltration scenario** 

- Colloidal species travel most rapidly through the UZ, and have the narrowest distribution of arrival times
- Conservative and sorbing species migrate more slowly
- Distribution of arrival times are much broader due to matrix diffusion
- Radioactive decay reduces the activity of many radionuclides in the UZ





### Results – Sensitivity to Infiltration Scenario Present-day climate

- Comparisons illustrate the TSPA abstraction model reproduces the process model results
- Infiltration uncertainty has a dramatic impact on transport model results

UZ Transport for Different Infiltration Scenarios <sup>99</sup>Tc, Black: FEHM Abstraction Model, Red: T2R3D Process Model





# Sensitivity to Active Fracture Model Parameter Gamma – Tc-99

Glacial-transition climate, mean infiltration scenario

- Flow model calibrations are relatively insensitive to several model parameters
- Sensitivity analyses are used to explore these uncertainties
- Active fracture model gamma parameter has a moderate impact on the breakthrough curve
- TSPA models use flow parameters at the conservative but reasonable end of the range





### Sensitivity to Active Fracture Model Parameter Gamma – Pu-242 Glacial-transition climate, mean infiltration scenario

- Active fracture model gamma parameter also has a moderate impact on the breakthrough curve of sorbing radionulcides
- Other flow model parameter results (not shown here)
- Other sensitivity analyses show low to moderate impact of flow model parameter uncertainties on the breakthrough curves





# Sensitivity to Diffusion Coefficient – Tc-99 and Pu-242

#### Present-day climate, mean infiltration scenario

- Diffusion coefficient has a large impact on the breakthrough curve
- For fracture transport, diffusion subdues the transport velocities of radionuclides traveling in the fractures by enabling migration into the slow-moving fluid in the rock matrix
- The impact of diffusion is most prevalent for sorbing radionuclides
- Uncertainty in diffusion coefficient is captured in TSPA model





## Sensitivity to Fracture-Matrix Interaction Model – Tc-99

#### Glacial-transition climate, mean infiltration scenario

- The conceptual model uncertainty associated with the diffusion to and from the fractures (fracture-matrix interaction model) has a significant impact on breakthrough curves
- Less early-time breakthrough is predicted with the (alternative) discrete fracture model because sharp gradients near the fracture are captured in this model





**Department of Energy** •Office of Civilian Radioactive Waste Management YMRobinson\_NWTRB\_020106.ppt

## Sensitivity to Fracture-Matrix Interaction Model – Pu-242

#### Glacial-transition climate, mean infiltration scenario

- The impact is even more dramatic for sorbing radionuclides
- The TSPA model conservatively uses the dual-k model
- A discrete fracture model (or equivalent) could be used in the future if the model is validated against field data
- Alternatively, a dual-k model with enhanced diffusion coefficients could be used





### **Fracture Versus Matrix Releases**

Tc-99, glacial-transition climate, mean infiltration scenario, point release

- Flow is fracturedominated at the repository horizon
- Matrix releases have significantly longer travel times because radionuclide must diffuse to the flowing fractures to travel rapidly
- Smaller diffusion coefficients yield longer travel times (the opposite of the fracture release case)





# Spatial Variability – Median Travel Time of Tc-99

**Glacial-transition climate, mean infiltration scenario** 

- Large variability in breakthrough curve depending on where the releases occur
  - Hydrogeologic variability
  - Percolation flux variability
- If only a few waste packages fail, this results in an uncertainty in travel time
- If most packages fail, this effect results in a spread in the distribution of arrival times
- All distributed release simulations presented earlier include this variability in the breakthrough curves







- Radionuclide transport in the Unsaturated Zone for TSPA is simulated considering all relevant transport processes
- Uncertainties that are most important to the travel times through the UZ are:
  - Infiltration rate
  - Diffusion model parameters
  - Diffusion conceptual model
- Uncertainties in flow model parameters have low to moderate impact on the travel times
- TSPA model takes a reasonably conservative approach for uncertainties not directly represented via parameter uncertainty distributions
- Matrix releases yield much longer travel times
  - Fracture releases lower D yields shorter travel times
  - Matrix releases lower D yields longer travel times
- Spatial variability of travel times results from different percolation fluxes and hydrogeology across the repository



