

VISION Contributions to DOE NWTRB Benchmark

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www.inl.gov

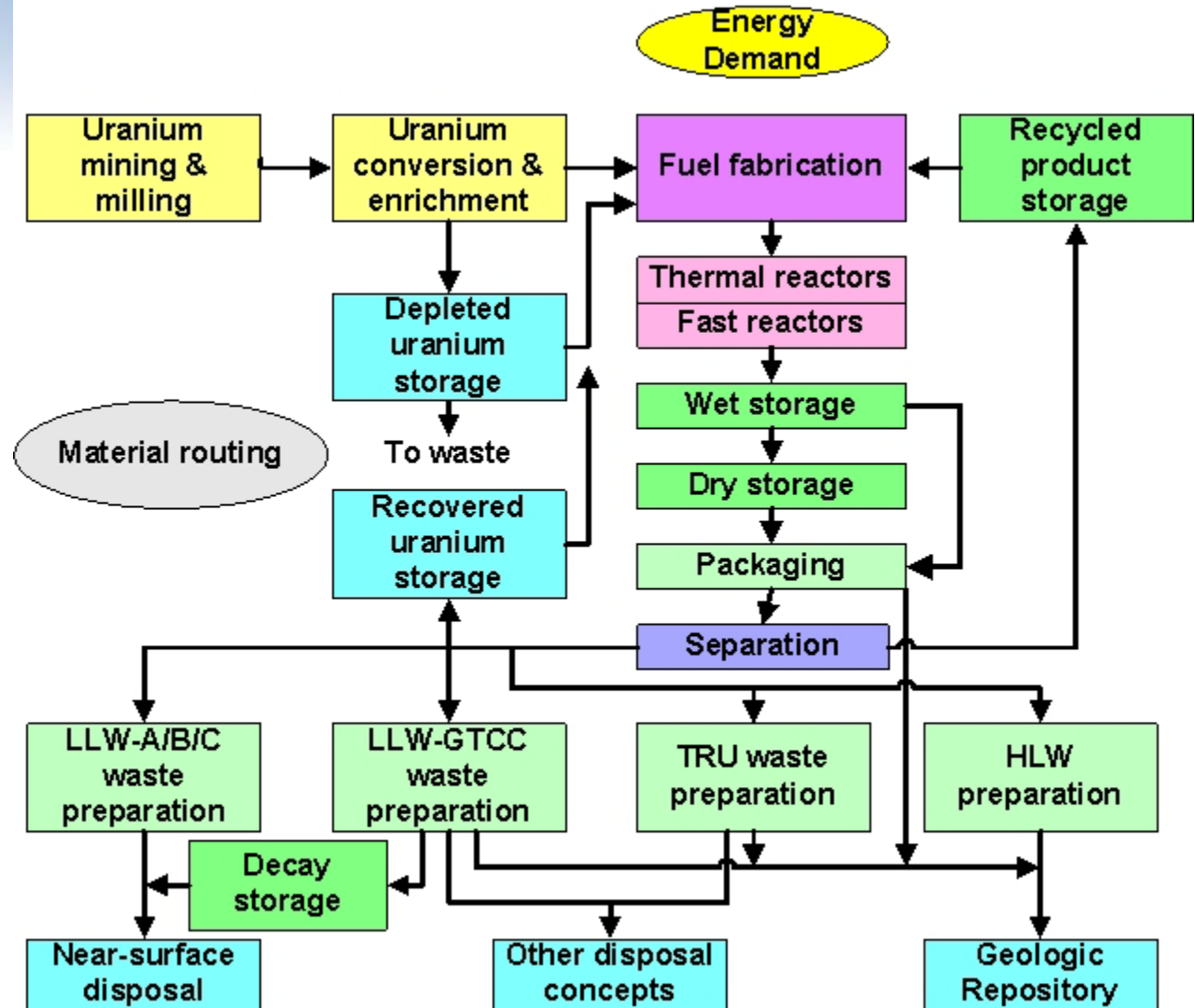


VISION is a tool for exploring advanced fuel cycle options

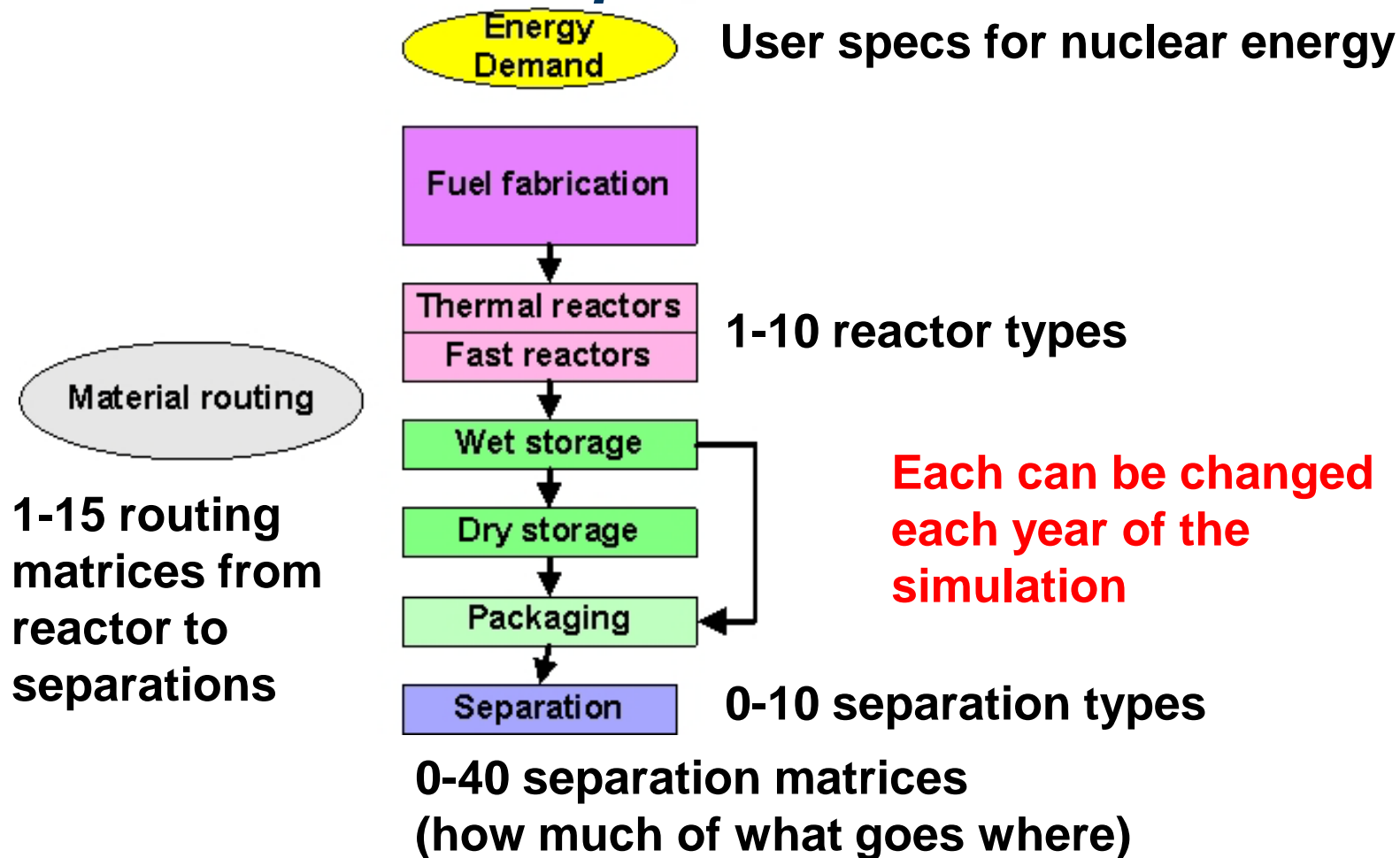
- Any U/TRU fuel cycle (probably Th/U in the future)
- Any U/TRU reactor
 - Models **types** of reactors, not individual reactors
- Any separation technology
- Technologies can be changed each year to study transitions
 - Nuclear electricity growth
 - Reactor mix
 - Fuel type and fuel fabrication capacity
 - Separation performance and capacity
 - Repository loading and receipt rate
 - Routing (reactor to separations, separations to fuel fab, etc.)
- 81 isotopes and groups of isotopes

VISION is a tool for exploring advanced fuel cycle options

NOTE: non-commercial wastes are not addressed



Some of VISION's capabilities



Benchmarks and comparisons

- Always take far longer and require far more iterations and specifications than anticipated.
 - Always start with general specs, then require additional iterations with more details to resolve differences in interpretations, etc.
- Cover only a portion of what a model requires as input.
- Cover only a portion of what a model provides.
 - Comparisons only possible on features common to all participating models
- Specifications often written to reflect the peculiarities of one of the models involved.
- Specifications never seem to be completely internally consistent, in part because real systems are more complex than any model.

Adjustments and interpretations

| VISION | Adjustment to specifications |
|--|---|
| Does not model hundreds of reactors | Can model ≤ 10 reactor “types”, each with fixed lifetime. In this case: PWR-40, PWR-60, BWR-40, BWR-60 |
| Has only one “legacy” retirement profile | Start calculations in 1960 so that existing reactors retire on time. |
| No true steady state develops (retirements, builds, isotope decay) | Obtain such results from as-stable-as-possible portion of a simulation |
| Does not do reactor physics, incorporates input/output fuel recipes. No recipes for re-enriched uranium. | Did not calculate re-enrichment cases. |
| Does not consider # of fuel assemblies | Did not calculate |

Adjustments and interpretations (continued)

- No two models handle “routing” the same way
- VISION is separation-centric
 - User specifies order in which each separation type “pulls” from used fuel inventories (rather than define “push” from used fuel)
 - Separation type 1 goes first, then 2, etc.
 - Most attention to waste resulting from flow through separations
- If used fuel direct to repository, VISION simply draws on fuel in the order that reactor types are defined in the input deck, in this case ...
 - PWR-40
 - PWR-60
 - BWR-40
 - BWR-60

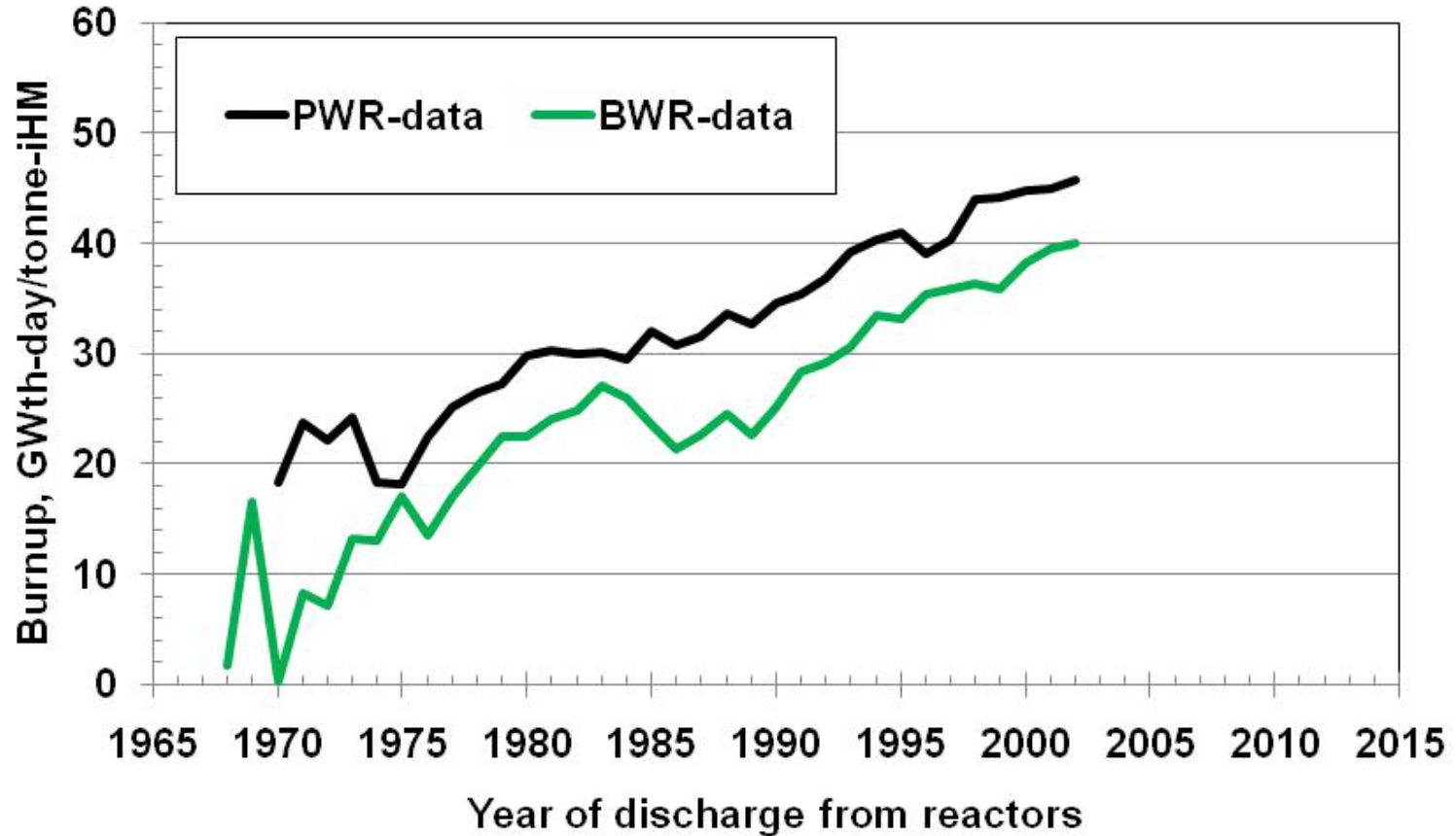
Adjustments and interpretations (continued)

- Can specify burnup or input enrichment, but not both
 - If LWR burnup, 4th order polynomial curve fit to PWR data provides input/output isotopic composition

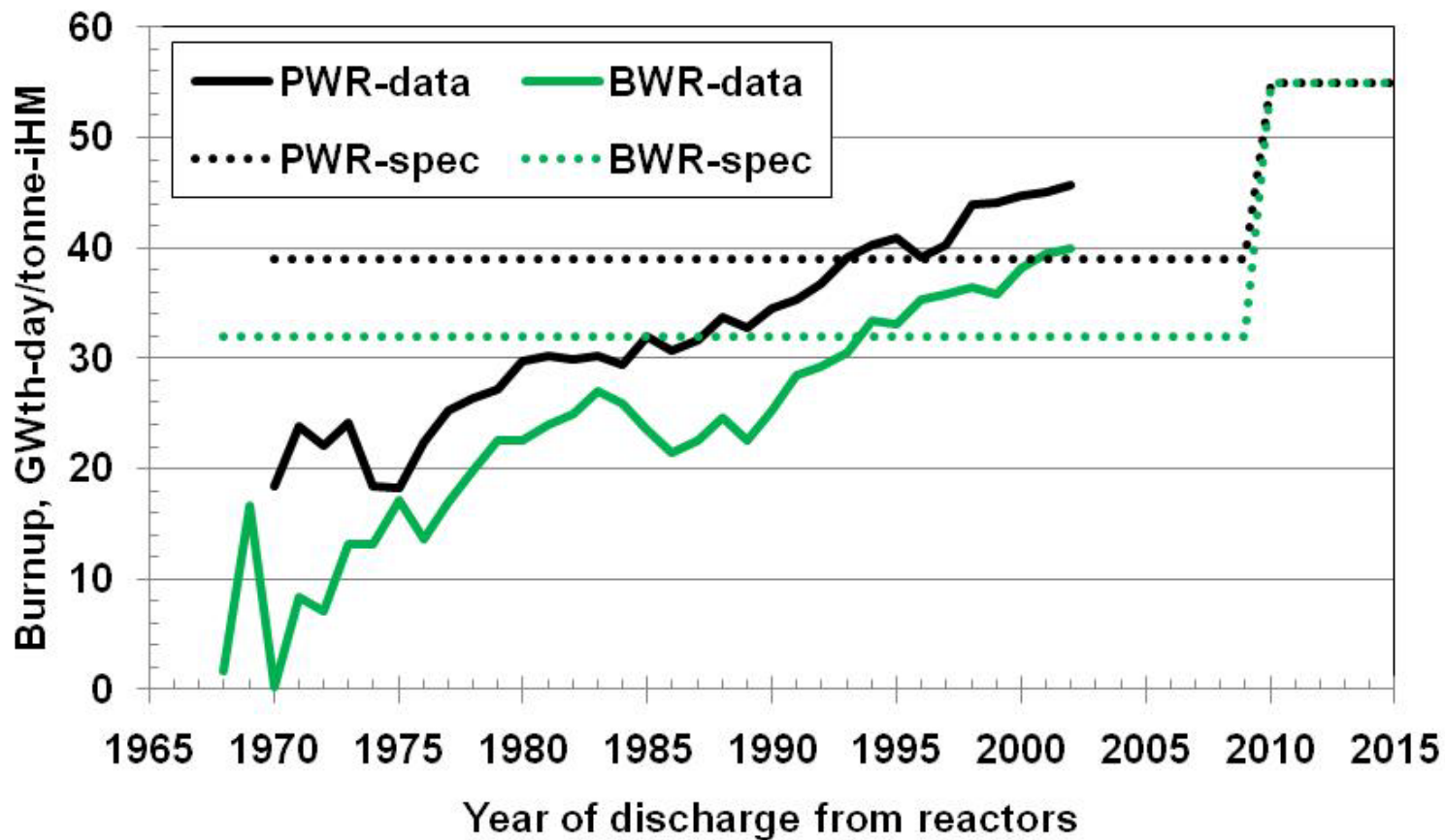
| | PWR | BWR | PWR | BWR |
|----------------|-------|-------|-------|-------|
| Spec GWd/tonne | 39 | 32 | 55 | 55 |
| U235 spec | 3.43% | 2.39% | 4.40% | 4.35% |
| U235 VISION | 3.38% | 2.89% | 4.59% | 4.59% |

- Known history of reactor start, constant 90% capacity factor, and specified constant burnup (“specs”) lead to **50,800 tonnes-iHM** in 2000 versus historical data of **42,600 tonnes-iHM**.
 - Keep reactor start data, hence reactor retirement, 90% capacity factor after 2000, specified burnup after 2010
 - Adjust pre-2010 burnup and capacity factor to match 42,600 tonnes-iHM (and find it then matches the spec in 2010).

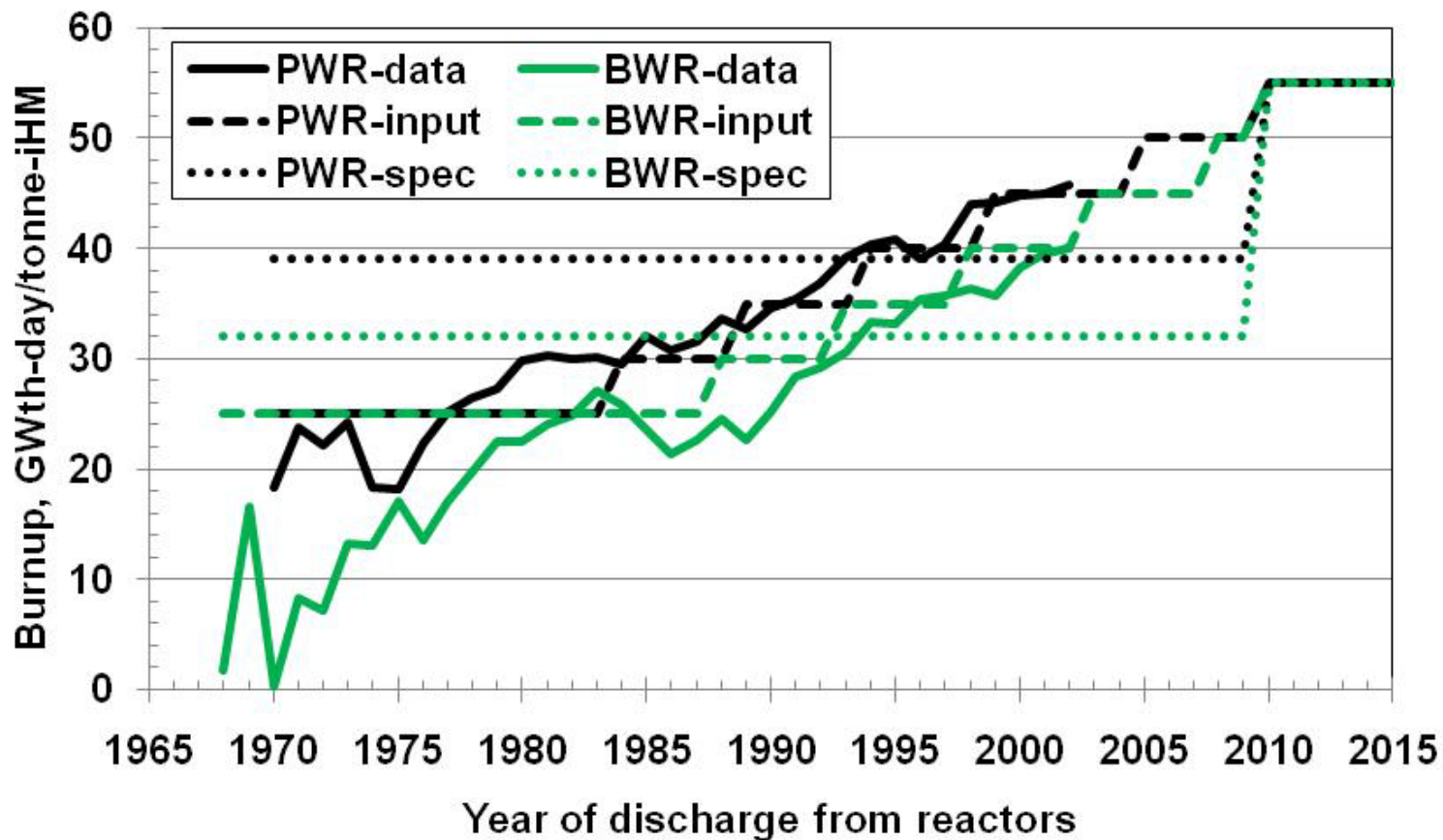
Adjusted burnup before 2010 to approximate the historical data



Adjusted burnup before 2010 to approximate the historical data



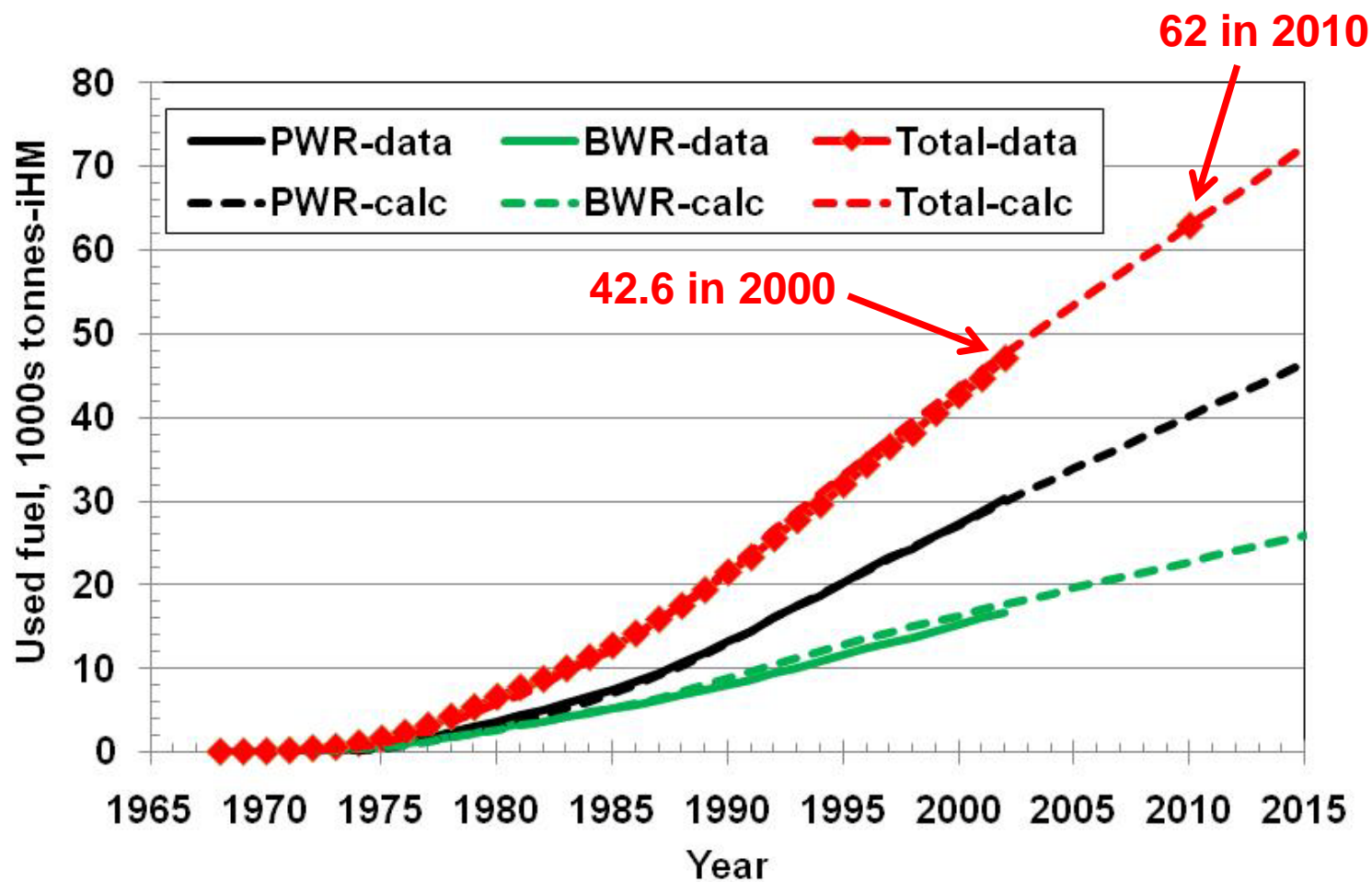
Adjusted burnup (“input”) before 2010 to approximate the historical data



Burnup, enrichment, capacity factor

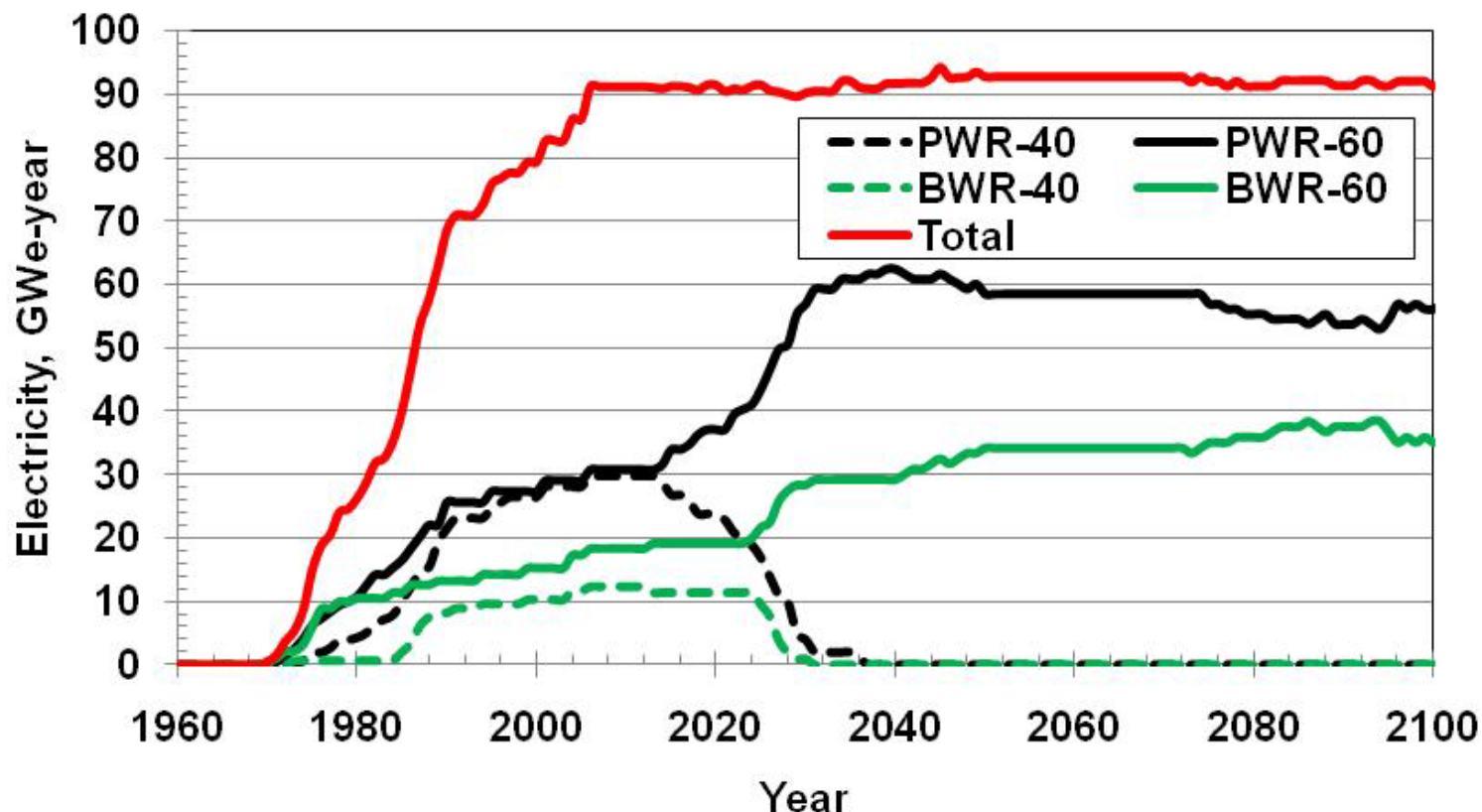
| | | | | | | | |
|------------------------|-------|-------|-------|-------|-------|-------|-------|
| When start/PWR | 1972 | 1984 | 1989 | 1994 | 1999 | 2005 | 2010 |
| When start/BWR | 1970 | 1988 | 1993 | 1998 | 2003 | 2008 | |
| Burnup | 25 | 30 | 35 | 40 | 45 | 50 | 55 |
| % U235 | 2.43% | 2.76% | 3.10% | 3.46% | 3.82% | 4.20% | 4.59% |
| Capacity factor PWR | 60% | 65% | 70% | 75% | 80% | 85% | 90% |
| Capacity factor BWR | | | 65% | 70% | 75% | | |

Used fuel inventory



Electricity

Current fleet modeled as either 40 or 60-year lifetime.
Per spec, all new reactors are 60-year lifetime.



Total electricity is not exactly constant as reactors are retiring and coming on line with differing unit sizes.

Phase 1 – End of 2009 used fuel inventory (tonnes)

| | PWR | BWR | Total |
|--------------------------------|---------------|---------------|---------------|
| Wet storage (≤ 10 years) | 13,431 | 6,615 | 20,046 |
| Dry storage (> 10 years) | 25,516 | 15,503 | 41,019 |
| Total | 38,948 | 22,118 | 61,065 |

Phase 1 – End of 2009 used fuel inventory (actinides, tonnes)

| | U232 | U233 | U234 | U235 | U236 | U238 | U-total |
|-----|-------|-------|-------|-------|-------|----------|----------|
| PWR | 0.0 | 0.0 | 0.5 | 267.3 | 157.8 | 36,622.8 | 37,048.4 |
| BWR | 0.0 | 0.0 | 0.2 | 151.8 | 79.5 | 20,921.1 | 21,152.7 |
| | Pu238 | Pu239 | Pu240 | Pu241 | Pu242 | Pu244 | Pu-total |
| PWR | 6.4 | 205.1 | 96.3 | 33.4 | 23.7 | 0.0 | 364.8 |
| BWR | 2.8 | 112.9 | 51.5 | 17.0 | 11.4 | 0.0 | 195.6 |
| | Np237 | Am | Cm | Bk-Cf | He | Other | MA-total |
| PWR | 16.6 | 26.0 | 1.3 | 0.0 | 0.0 | 0.0 | 43.9 |
| BWR | 8.0 | 13.5 | 0.5 | 0.0 | 0.0 | 0.0 | 22.0 |

58,800 tonnes (>96% of used fuel) of recyclable material

Phase 1 – End of 2009 used fuel inventory (fission products, tonnes)

| | H3 | C14 | C other | Kr81 | Kr85 | Inert gas- other | | | | | Subtotal | |
|-----|-------|----------|----------|-------------------|-------|---------------------|----------|--------------|-------|--------------|----------|-----------------------------------|
| PWR | 0.00 | 0.00 | 0.00 | 0.00 | 0.55 | 244.38 | | | | | 244.93 | Volatiles |
| BWR | 0.00 | 0.00 | 0.00 | 0.00 | 0.27 | 121.54 | | | | | 121.81 | |
| | Tc99 | Tc other | I129 | Halogen- other | | | | | | | Subtotal | |
| PWR | 33.47 | 0.01 | 7.85 | 3.68 | | | | | | | 45.03 | Long-lived |
| BWR | 17.06 | 0.01 | 4.00 | 1.88 | | | | | | | 22.95 | |
| | Rb | Sr90 | Sr-other | Cs134 | Cs135 | Cs137 | Cs-other | Ba | | | Subtotal | |
| PWR | 15.05 | 17.91 | 15.96 | 0.82 | 16.57 | 41.60 | 48.05 | 76.80 | | | 232.75 | Heat |
| BWR | 7.57 | 8.94 | 8.09 | 0.36 | 9.00 | 20.71 | 24.65 | 38.34 | | | 117.68 | |
| | Zr93 | Zr95 | Zr-other | Ru106 | Pd107 | Se79 | Cd113m | Sn126 | Sb125 | TM- other | Subtotal | |
| PWR | 30.92 | 0.08 | 123.68 | 0.39 | 10.18 | 0.25 | 0.01 | 1.21 | 0.09 | 351.54 | 518.35 | Transition metals (TM) |
| BWR | 15.56 | 0.04 | 62.03 | 0.19 | 5.07 | 0.13 | 0.00 | 0.61 | 0.05 | 175.75 | 259.43 | |
| | Ce144 | Pm147 | Sm147 | Sm151 | Eu154 | Eu155 | Ho166m | LA- other | | | Subtotal | |
| PWR | 0.54 | 0.72 | 2.63 | 0.61 | 0.81 | 0.19 | 0.00 | 443.94 | | | 449.44 | Lanthanides (LA) |
| BWR | 0.27 | 0.36 | 1.33 | 0.35 | 0.37 | 0.09 | 0.00 | 222.64 | | | 225.40 | |

2,238 tonnes of fission products

Phase 2 – 2010 to 2100 used fuel inventory (tonnes)

| 2100 inventory | PWR | BWR | Total |
|--------------------------------|----------------|---------------|----------------|
| Wet storage (≤ 10 years) | 13,405 | 9,141 | 22,546 |
| Dry storage (> 10 years) | 143,031 | 79,542 | 222,573 |
| Total | 156,436 | 88,683 | 245,119 |

| 2010 to 2100 | PWR | BWR | Total |
|---------------------|------------|------------|--------------|
| Discharged | 117,488 | 66,565 | 184,054 |

Discharge ~2000 tonnes/year

Phase 2 – 2010 to 2100 used fuel discharged (actinides, tonnes)

| | U232 | U233 | U234 | U235 | U236 | U238 | U-total |
|-----|-------|-------|-------|-------|-------|-----------|-----------|
| PWR | 0.0 | 0.0 | 5.2 | 846.7 | 712.2 | 107,736.8 | 109,301.0 |
| BWR | 0.0 | 0.0 | 2.9 | 479.7 | 403.6 | 61,039.5 | 61,925.7 |
| | Pu238 | Pu239 | Pu240 | Pu241 | Pu242 | Pu244 | Pu-total |
| PWR | 38.9 | 691.8 | 352.2 | 100.1 | 112.3 | 0.0 | 1,295.3 |
| BWR | 22.2 | 392.0 | 199.5 | 58.4 | 63.6 | 0.0 | 735.7 |
| | Np237 | Am | Cm | Bk-Cf | He | Other | MA-total |
| PWR | 84.9 | 140.4 | 7.6 | 0.0 | 0.4 | 0.0 | 233.3 |
| BWR | 48.1 | 77.9 | 4.4 | 0.0 | 0.2 | 0.0 | 130.6 |

174,000 tonnes (>94% of used fuel) of recyclable material

Phase 2 – 2010 to 2100 used fuel discharged (fission products, tonnes)

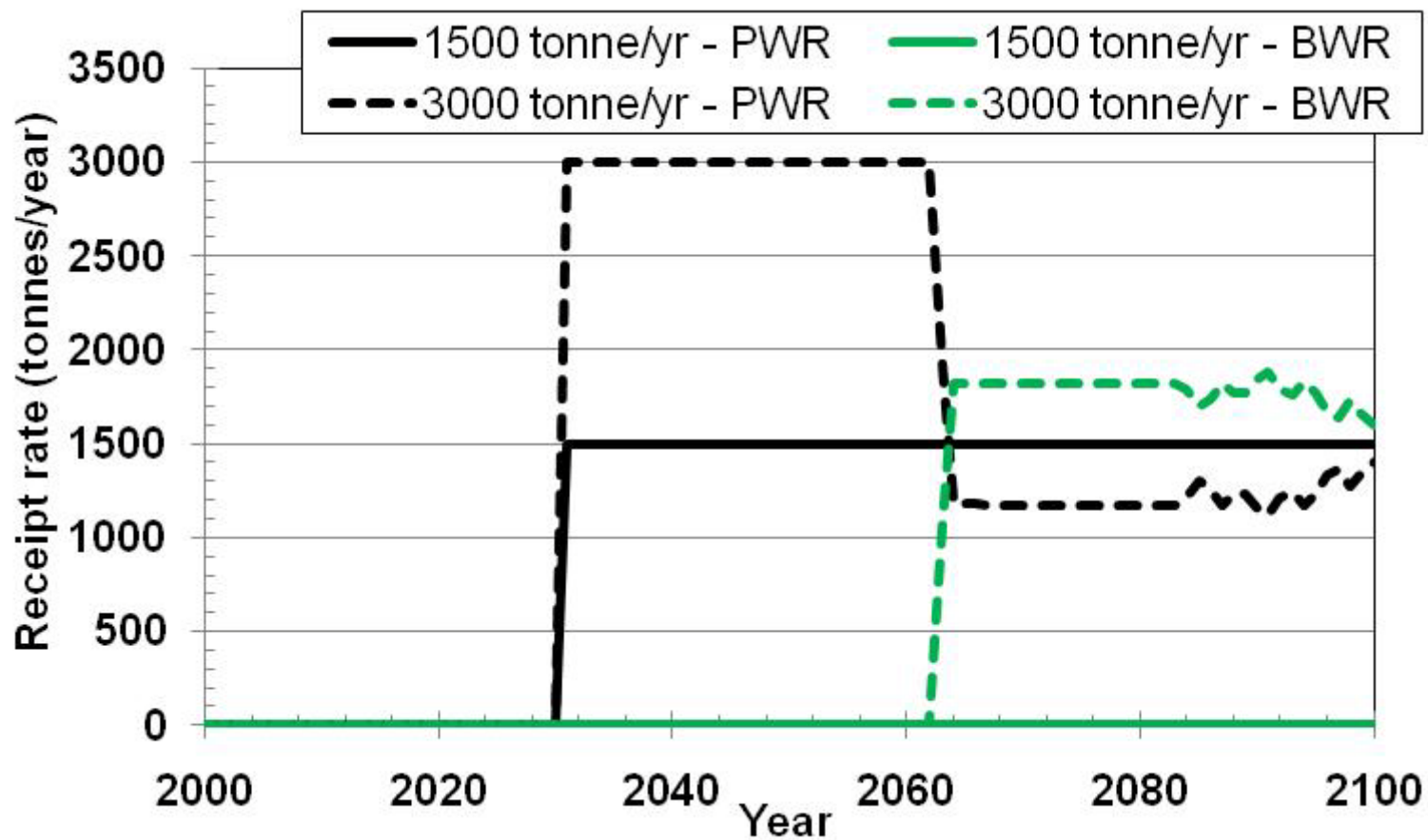
| | H3 | C14 | C other | Kr81 | Kr85 | Inert gas- other | | | | | | Subtotal | |
|-----|--------|----------|----------|-------------------|-------|---------------------|----------|--------------|-------|--------------|--|----------|-----------------------------------|
| PWR | 0.00 | 0.00 | 0.01 | 0.00 | 1.62 | 1085.38 | | | | | | 1,087.02 | Volatiles |
| BWR | 0.00 | 0.00 | 0.00 | 0.00 | 0.96 | 615.04 | | | | | | 616.01 | |
| | Tc99 | Tc other | I129 | Halogen- other | | | | | | | | Subtotal | |
| PWR | 143.35 | 0.04 | 34.11 | 15.49 | | | | | | | | 192.99 | Long-lived |
| BWR | 81.23 | 0.02 | 19.33 | 8.78 | | | | | | | | 109.36 | |
| | Rb | Sr90 | Sr-other | Cs134 | Cs135 | Cs137 | Cs-other | Ba | | | | Subtotal | |
| PWR | 68.23 | 69.57 | 69.93 | 0.20 | 82.11 | 162.38 | 202.26 | 380.82 | | | | 1,035.50 | Heat |
| BWR | 38.62 | 39.95 | 39.63 | 0.30 | 46.54 | 93.18 | 114.61 | 214.45 | | | | 587.27 | |
| | Zr93 | Zr95 | Zr-other | Ru106 | Pd107 | Se79 | Cd113m | Sn126 | Sb125 | TM- other | | Subtotal | |
| PWR | 137.46 | 0.00 | 566.74 | 0.00 | 45.18 | 1.13 | 0.03 | 5.31 | 0.04 | 1576.57 | | 2,332.46 | Transition metals (TM) |
| BWR | 77.90 | 0.01 | 320.63 | 0.07 | 25.60 | 0.64 | 0.02 | 3.01 | 0.04 | 893.29 | | 1,321.20 | |
| | Ce144 | Pm147 | Sm147 | Sm151 | Eu154 | Eu155 | Ho166m | LA- other | | | | Subtotal | |
| PWR | -0.02 | 0.19 | 12.24 | 2.36 | 2.48 | 0.34 | 0.00 | 1,993.16 | | | | 2,010.75 | Lanthanides (LA) |
| BWR | 0.08 | 0.23 | 6.93 | 1.34 | 1.50 | 0.23 | 0.00 | 1,129.12 | | | | 1,139.42 | |

10,400 tonnes of fission products

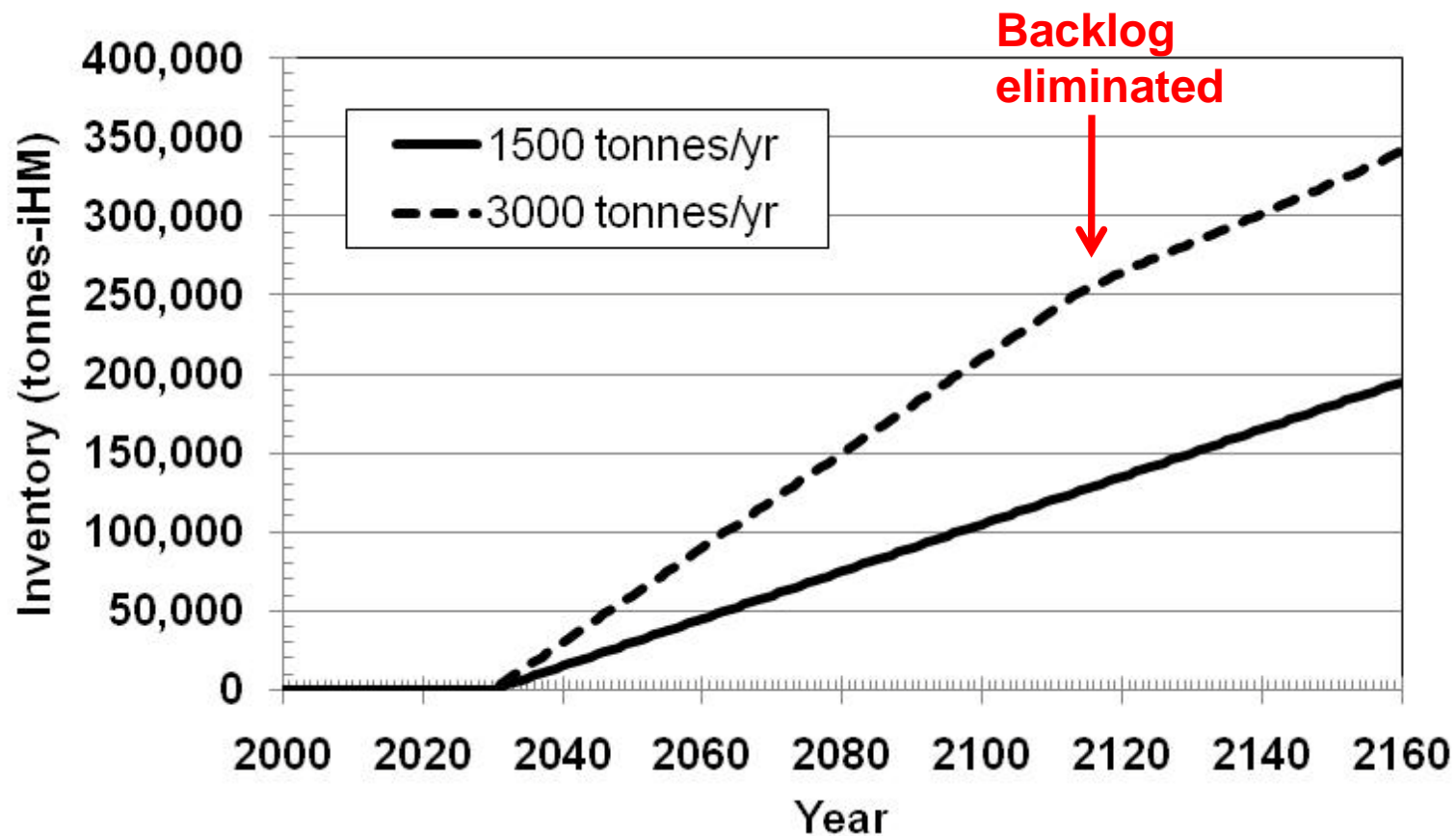
Phase 3 – Repository, but no recycling

- Cases specified, both start repository in 2030
 - 1500 tonnes/year – backlog continues to grow
 - 3000 tonnes/year – backlog eliminated in 2115
 - Per spec, there is no limit to repository capacity
- VISION sends waste (once old enough) to repository based on the order of reactors defined in the input file
 - In this case, takes PWR-40, then PWR-60, BWR-40, PWR-60
- Isotopic data in spreadsheet

Phase 3 – Repository receipt rate (tonnes/yr)



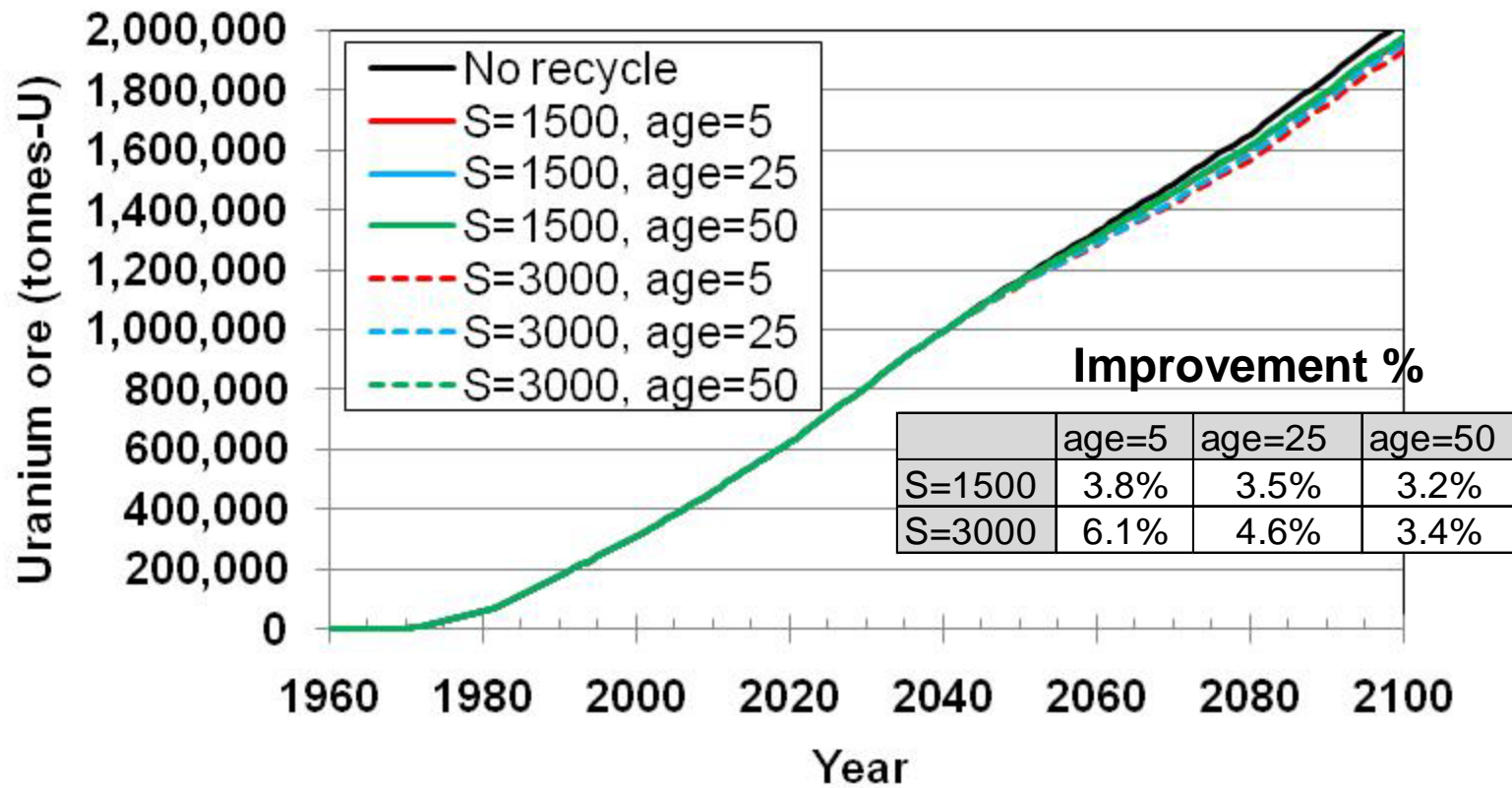
Phase 3 – repository inventory (tonnes)



Phase 4 – Recycling, but no repository

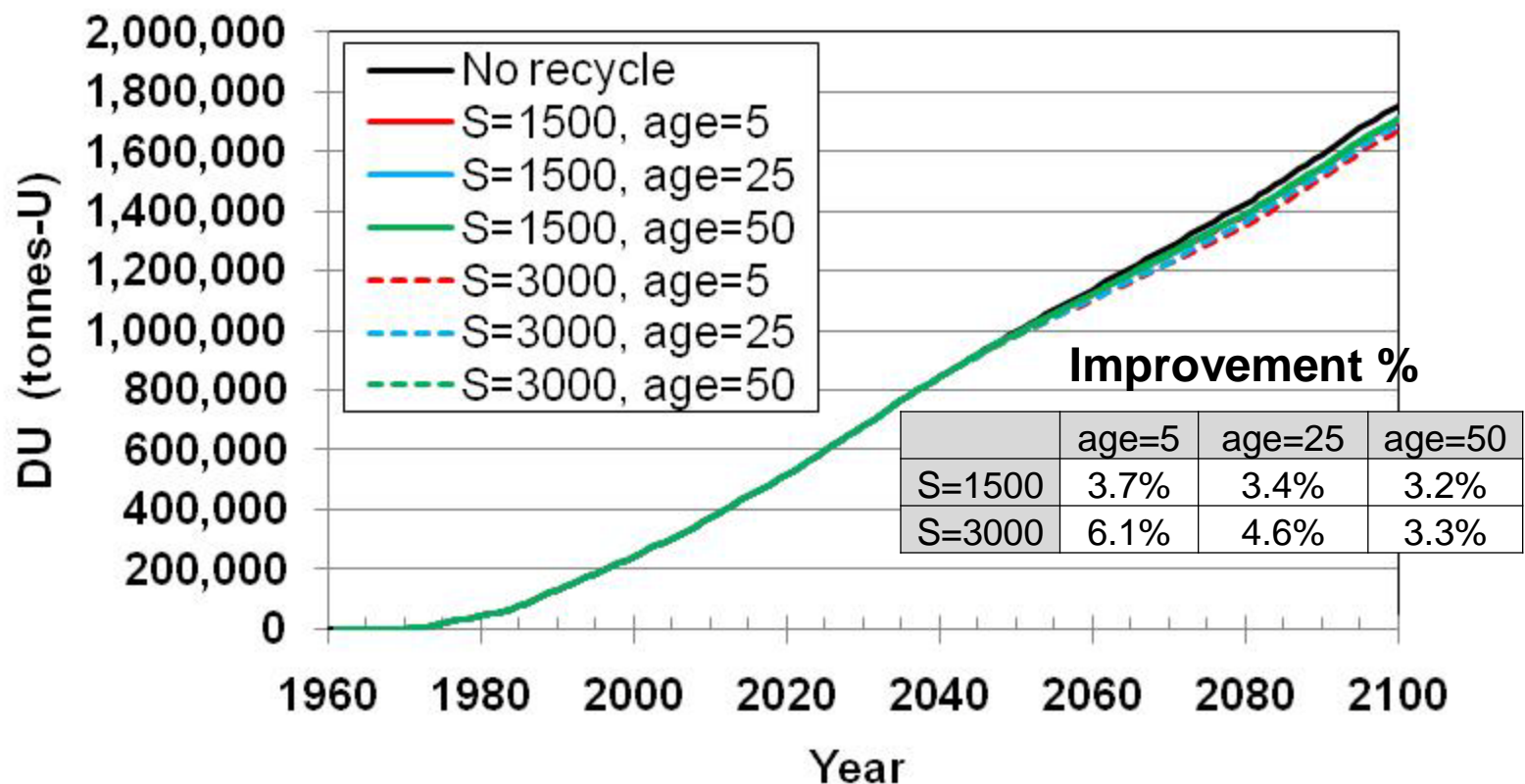
- MOX-RU-Pu, fuel recipe in Library has burnup of 50 GWth-day/tonne
 - Recycles RU from separated LWR UOX
 - Recycles Pu (11% of fresh MOX)
 - Best possible MOX-Pu uranium improvement (1 recycle) is 14%
 - x 64% of the fuel x 2/3 of the time (2010-2100) → **6.0%**
- Minimum aging before separation
 - **5 years, 25 years, 50 years**
- Separation cases specified, both start in 2040, tonnes-iHM
 - 1500 t/year – backlog grows, U savings **3.8%, 3.5%, 3.2%**
 - 3000 t/year – backlog gone in 2118, U savings **6.1%, 4.6%, 3.4%**
- Per spec, BWR fuel not recycled
- Isotopic data in spreadsheet

Phase 4 – Recycling – uranium ore consumption



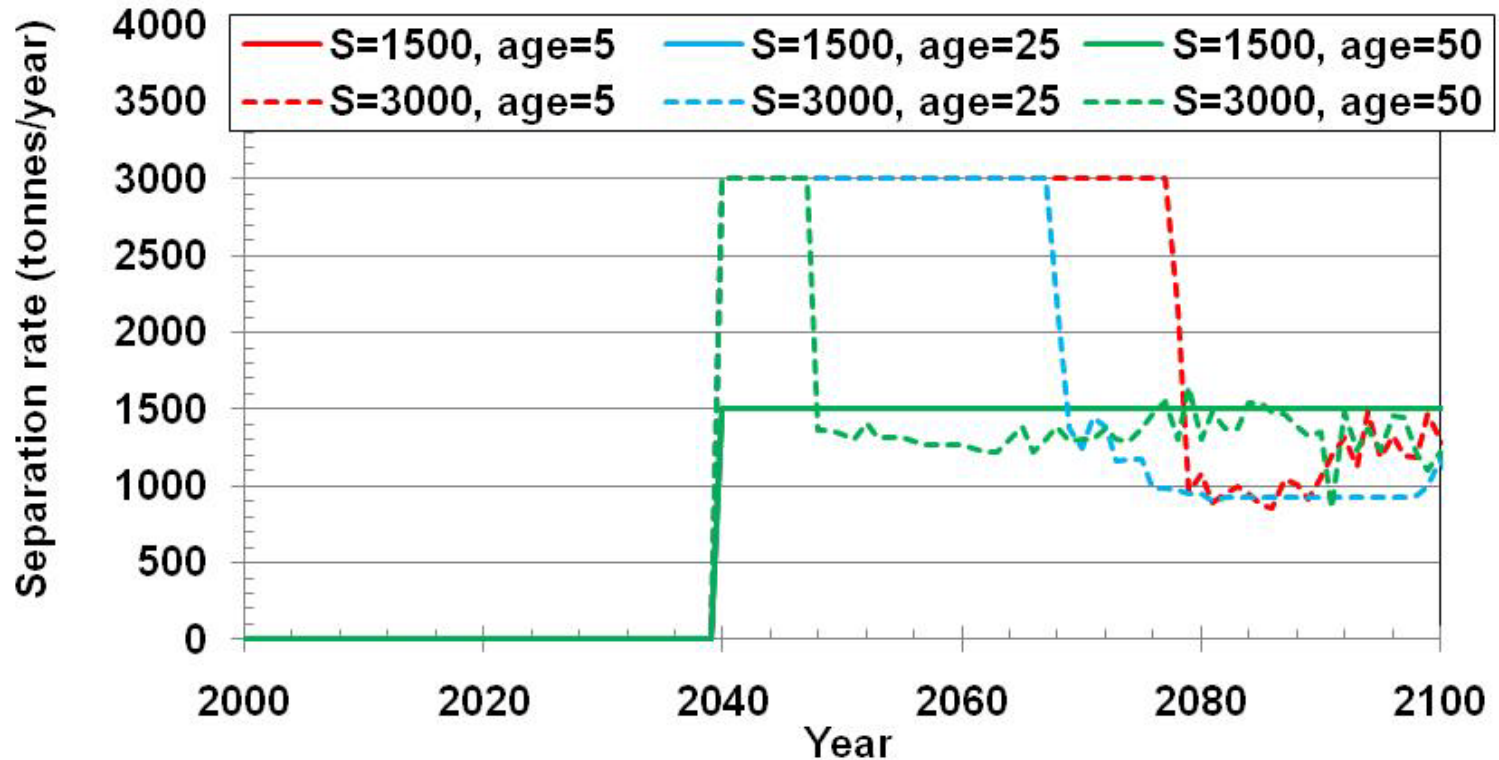
Unit is tonnes of uranium metal in the ore

Phase 4 – Recycling – depleted uranium generated



Unit is tonnes of uranium metal in the ore

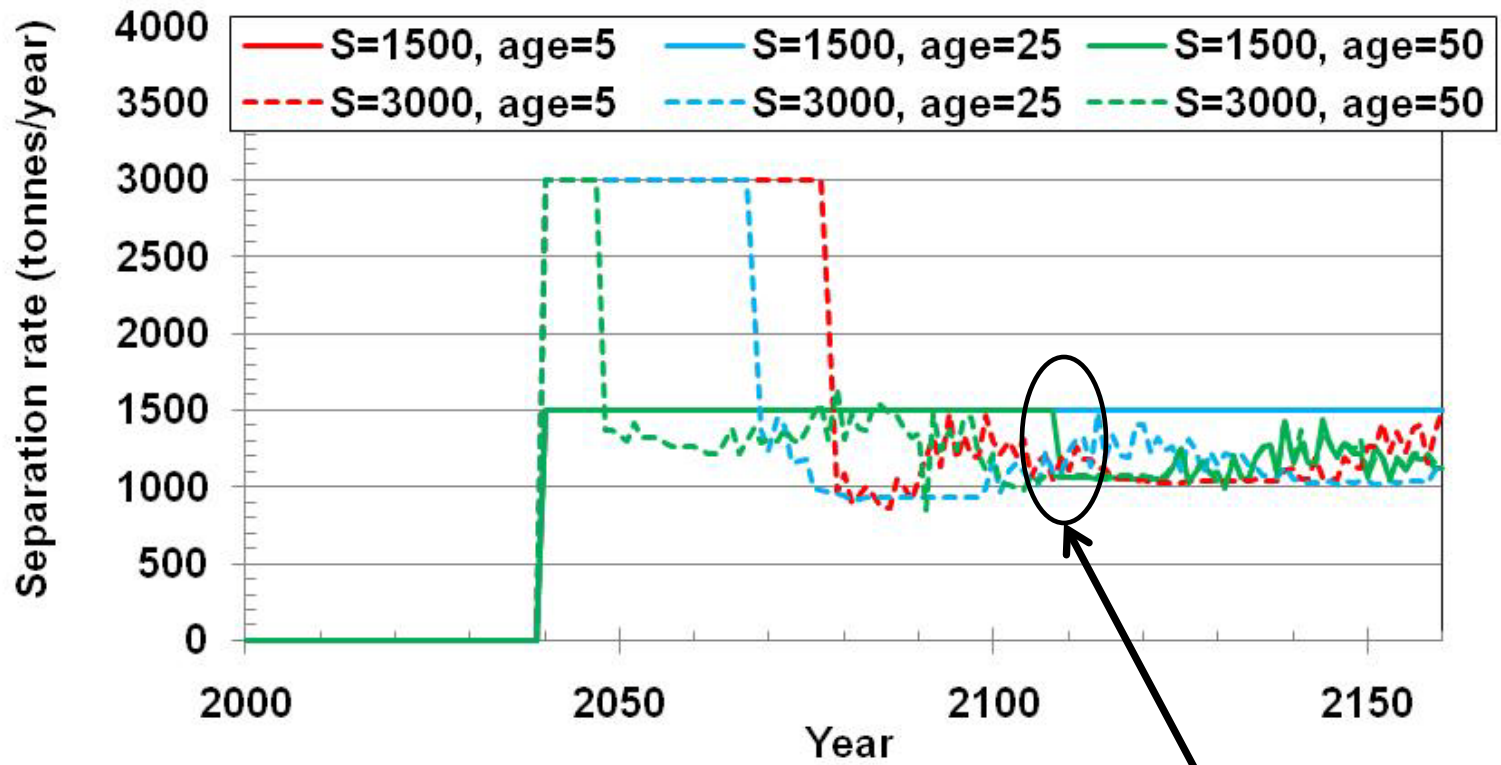
Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



At 1500 tonne/year, backlog continues

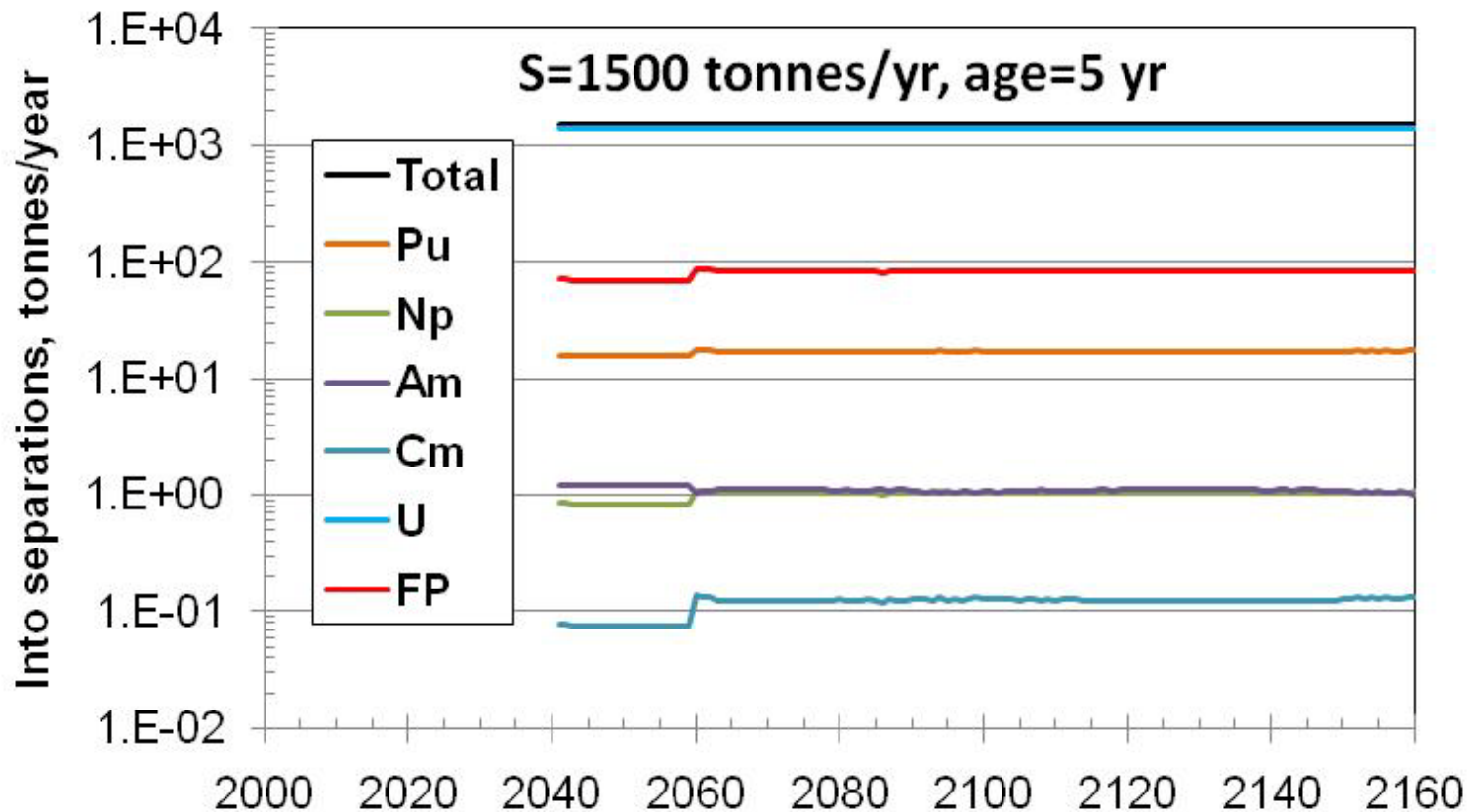
At 3000 tonne/year, backlog gone in 2048 (age=50), 2068 (age=25), 2078 (age=5)

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



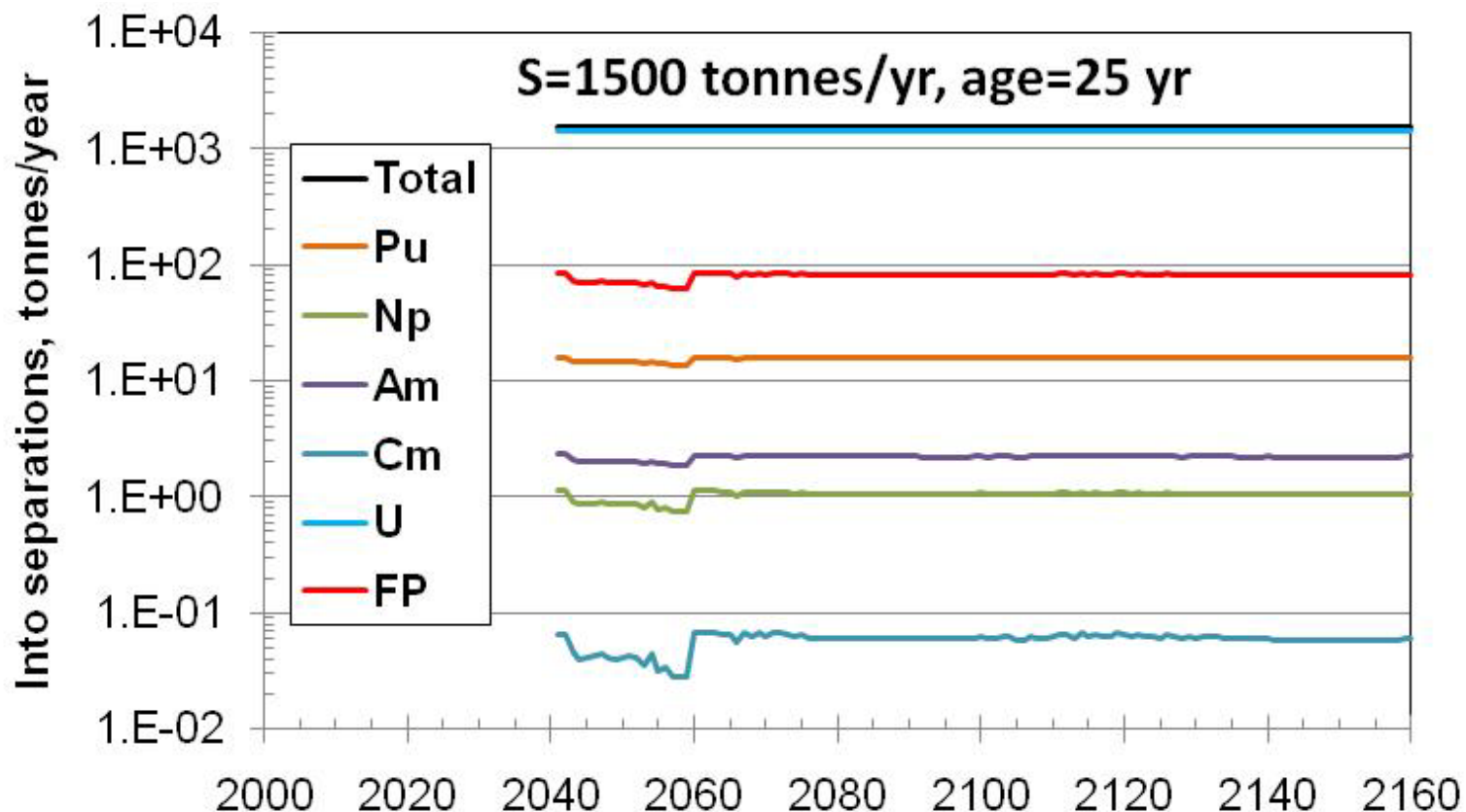
At 1500 tonne/year, age=50, backlog gone in 2109

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel

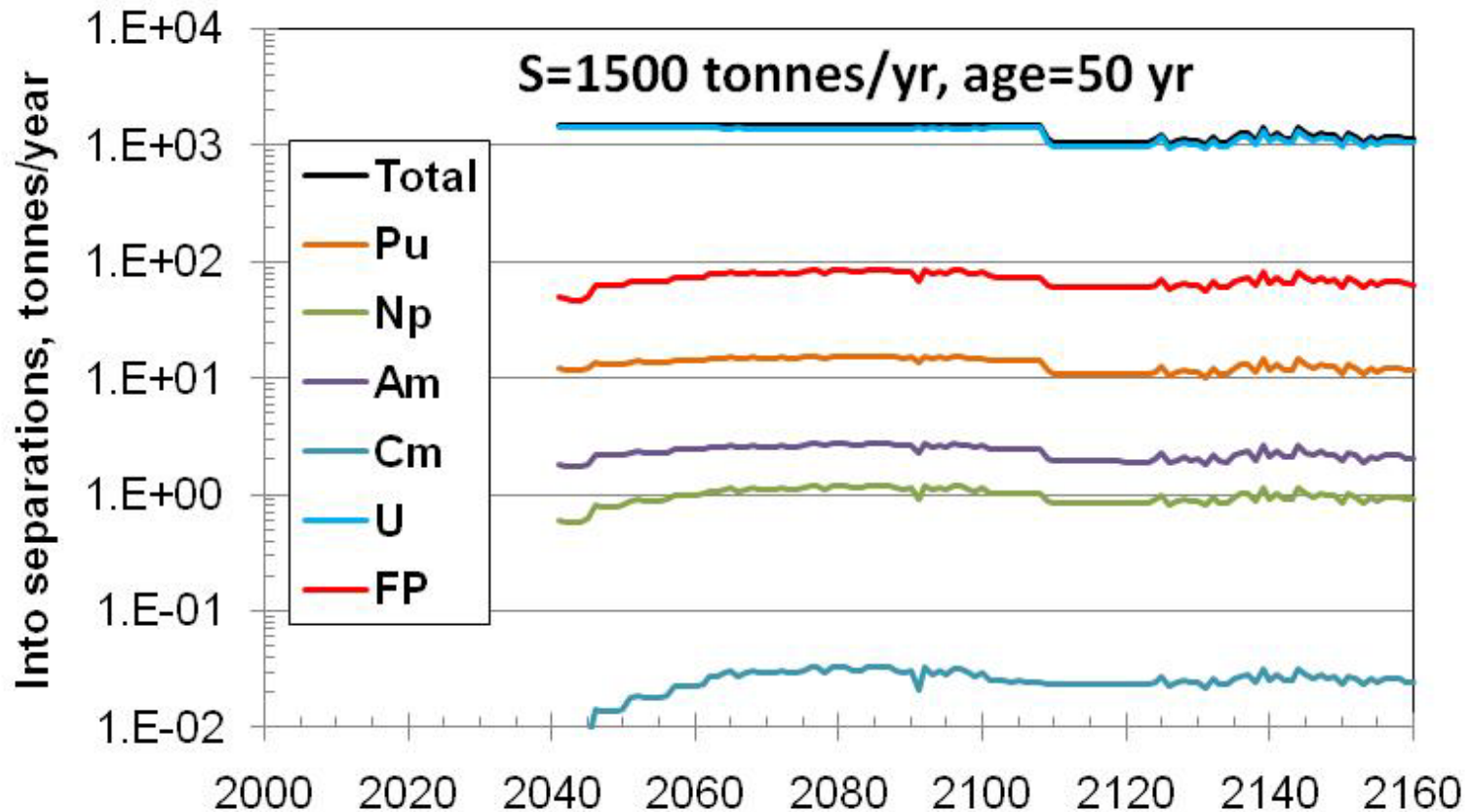


Two effects at work: isotope decay and different BU fuel used at different times

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel

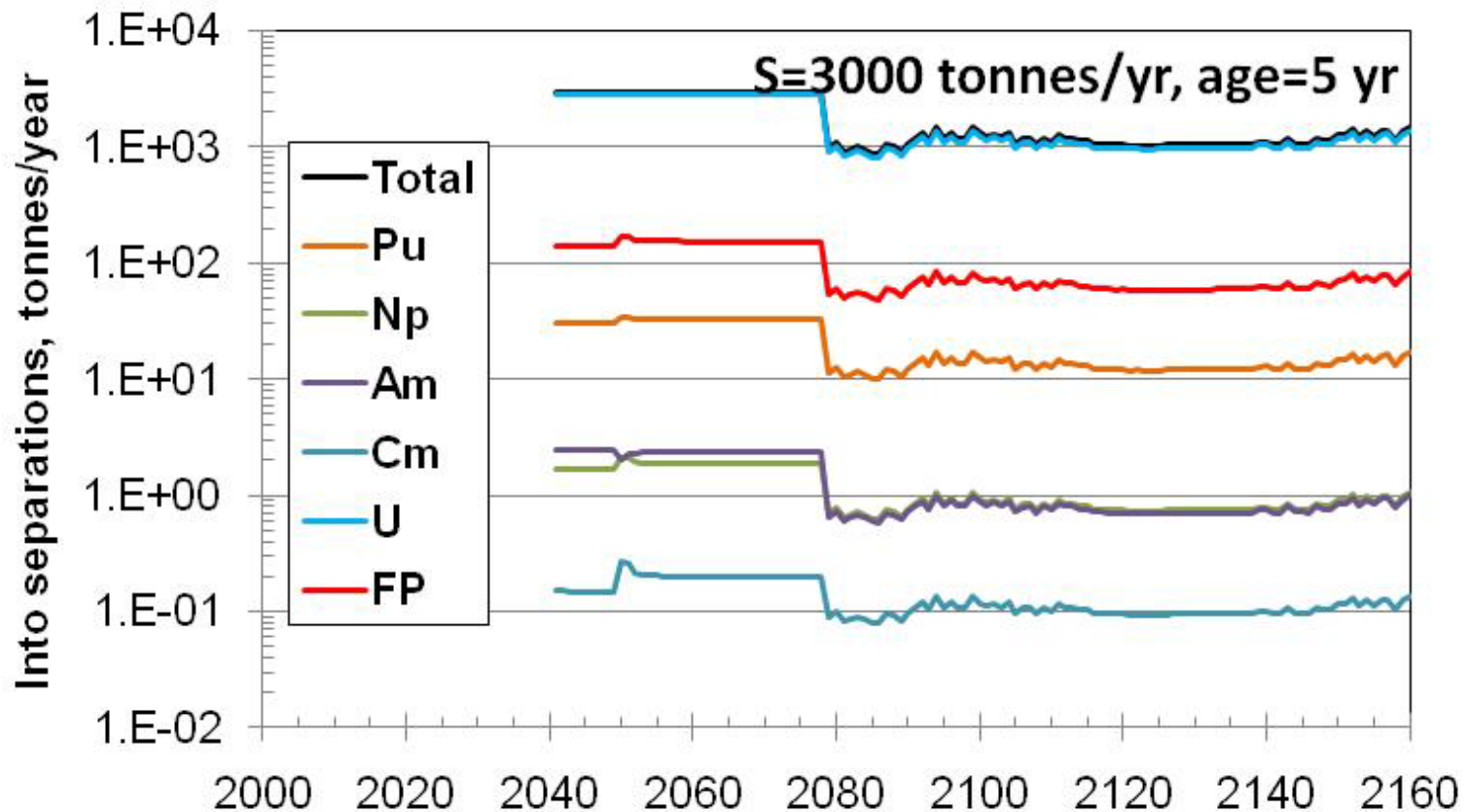


Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



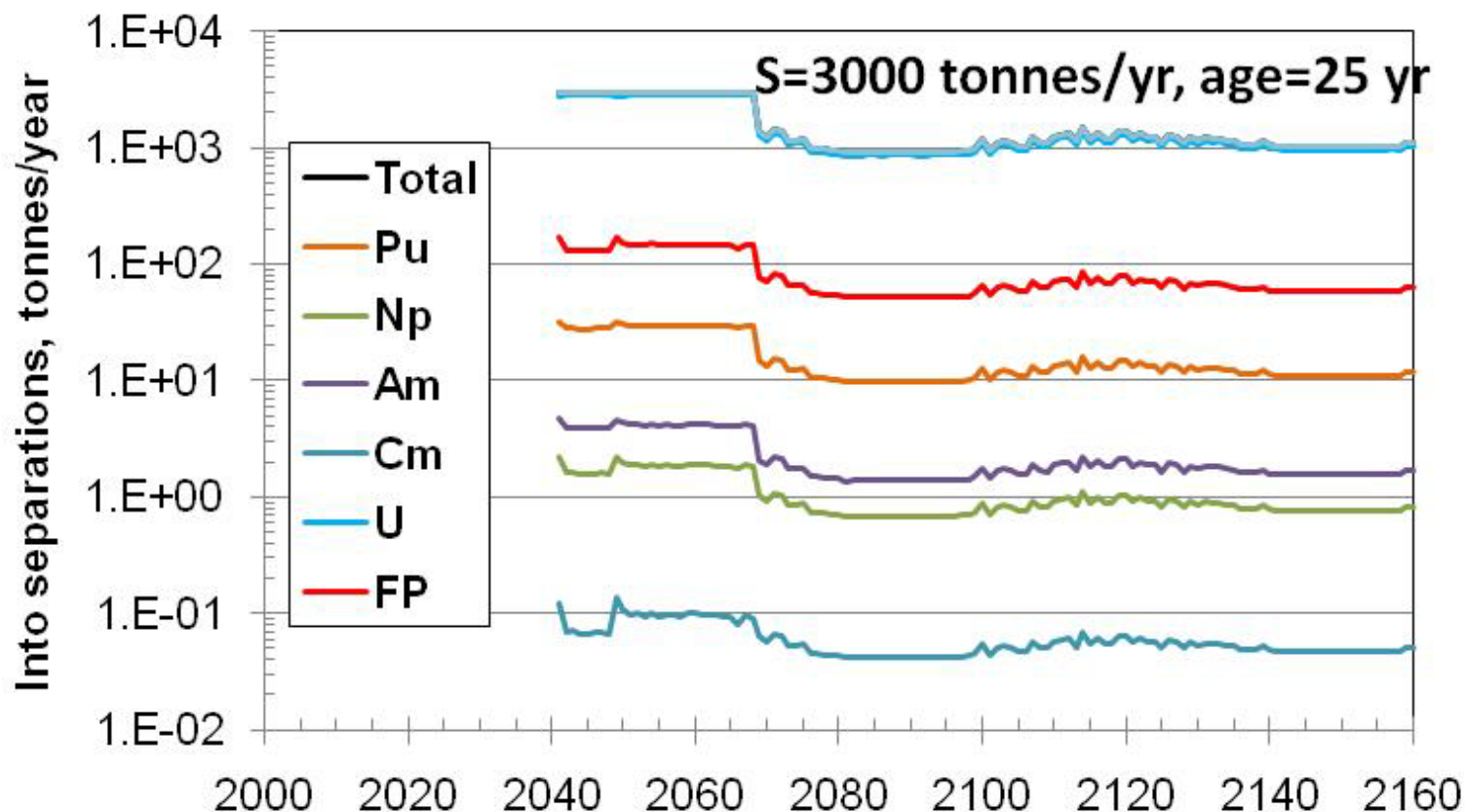
Dip in 2109 is when backlog worked off

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



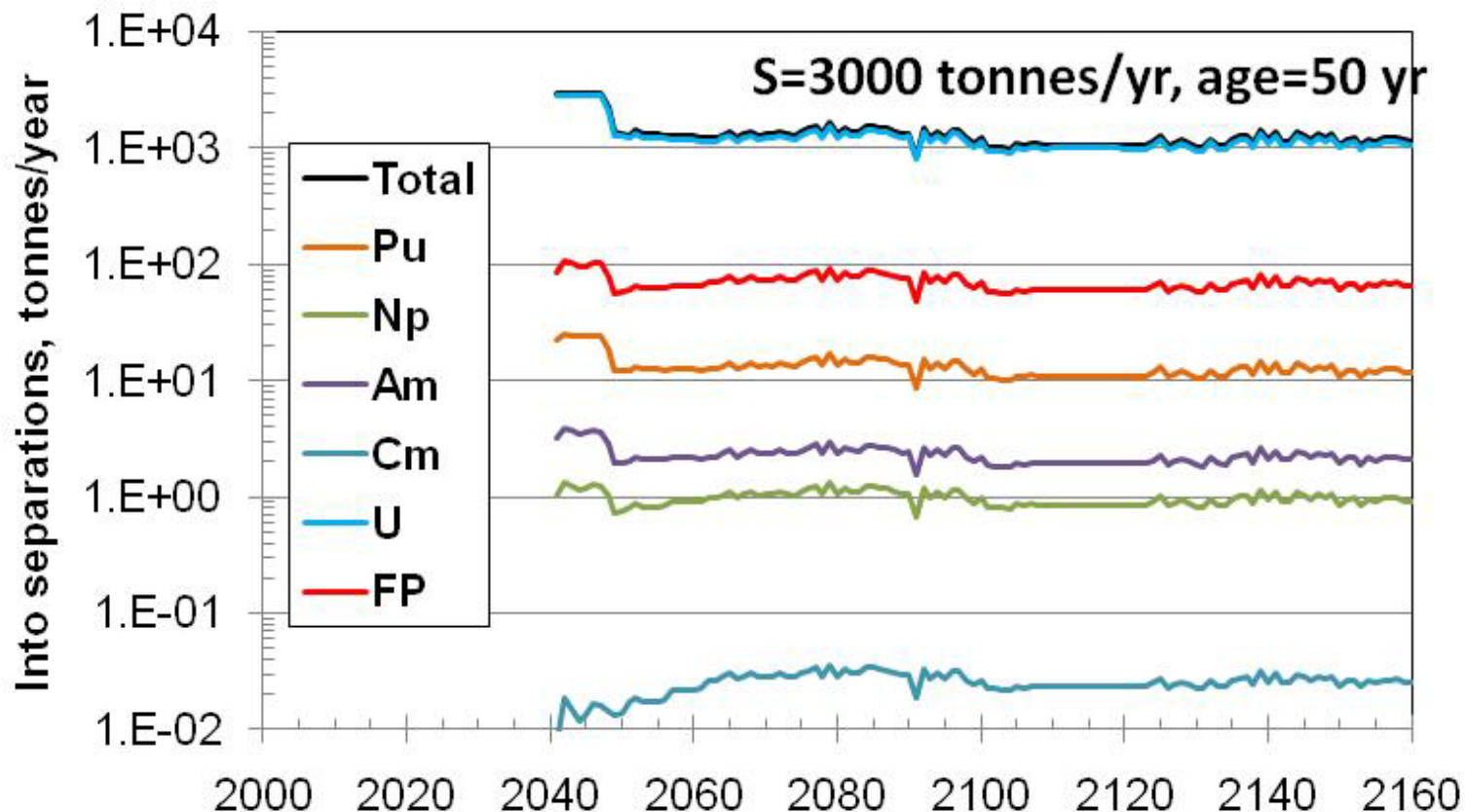
Dip in 2078 is when backlog worked off

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



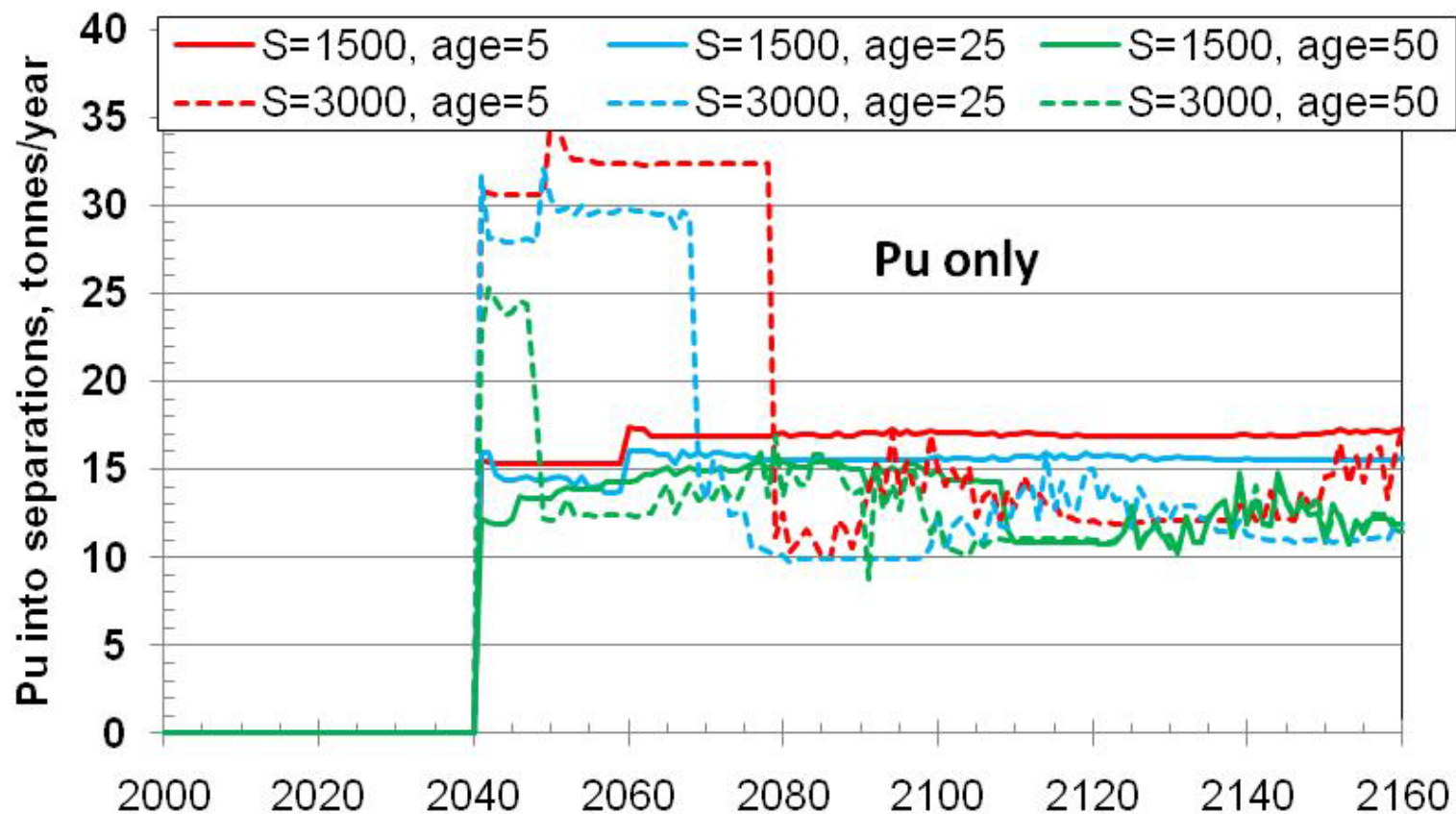
Dip in 2068 is when backlog worked off

Phase 4 – Recycling rate (tonnes/yr) of PWR fuel

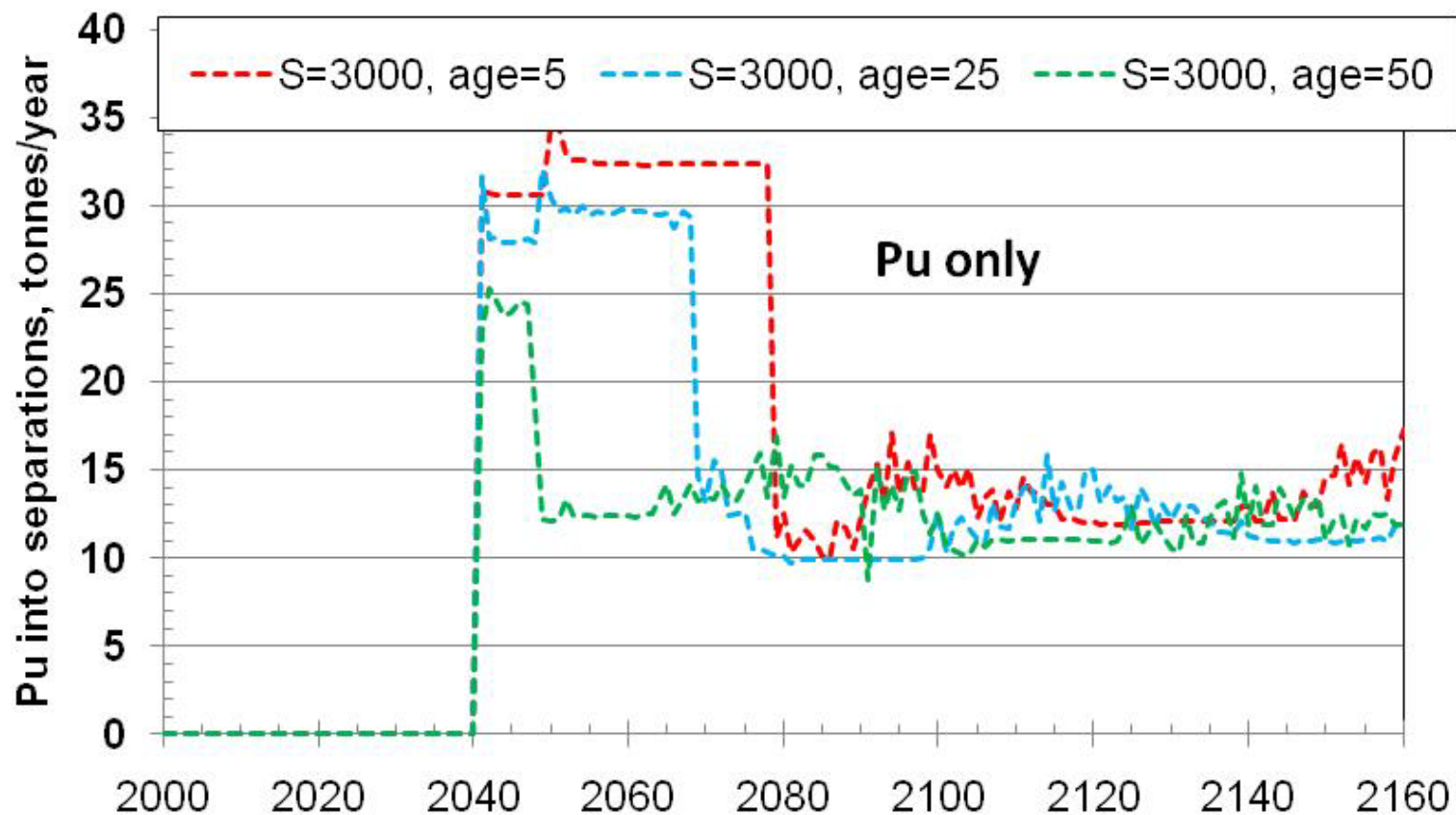


Dip in 2048 is when backlog worked off

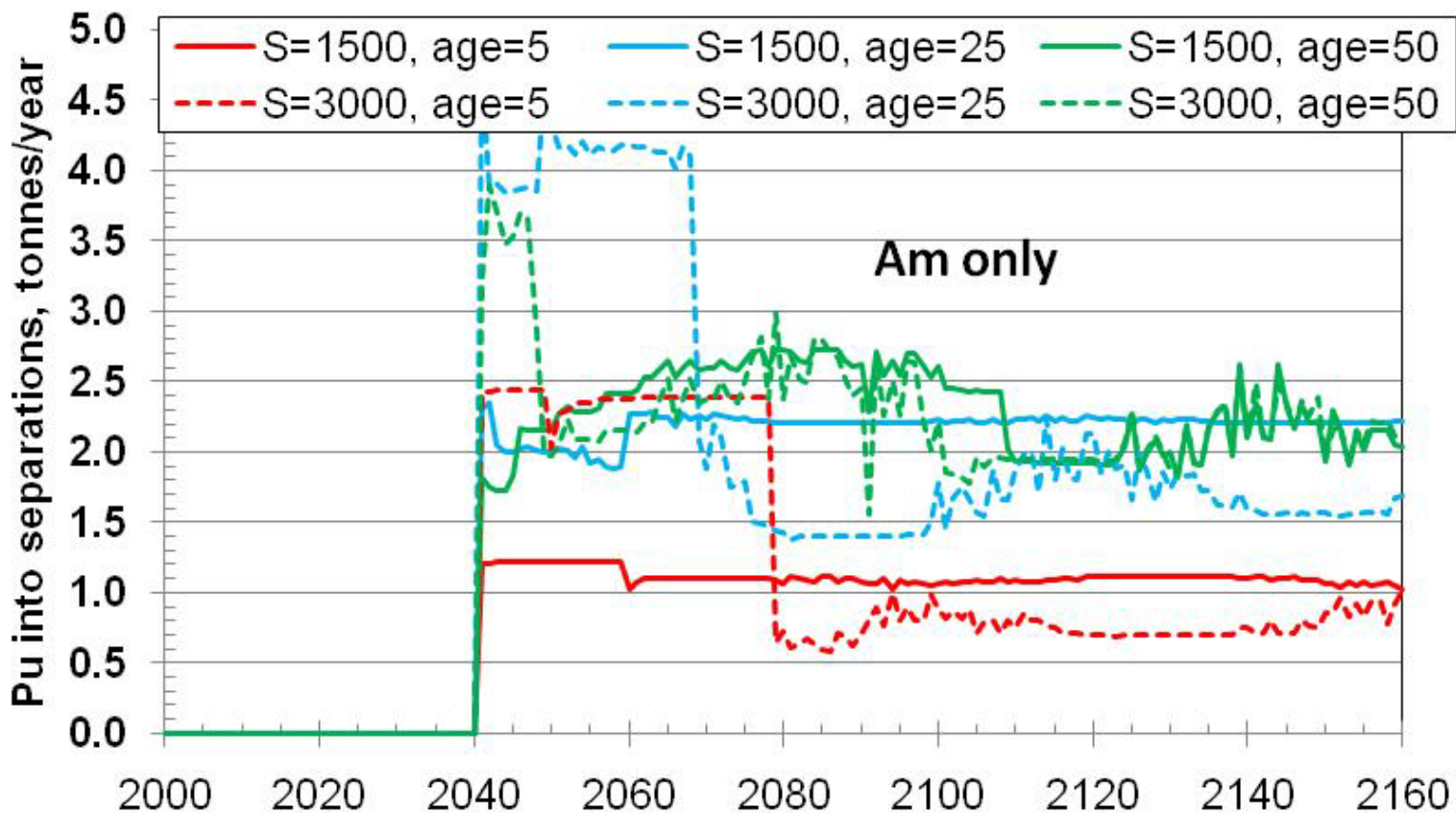
Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



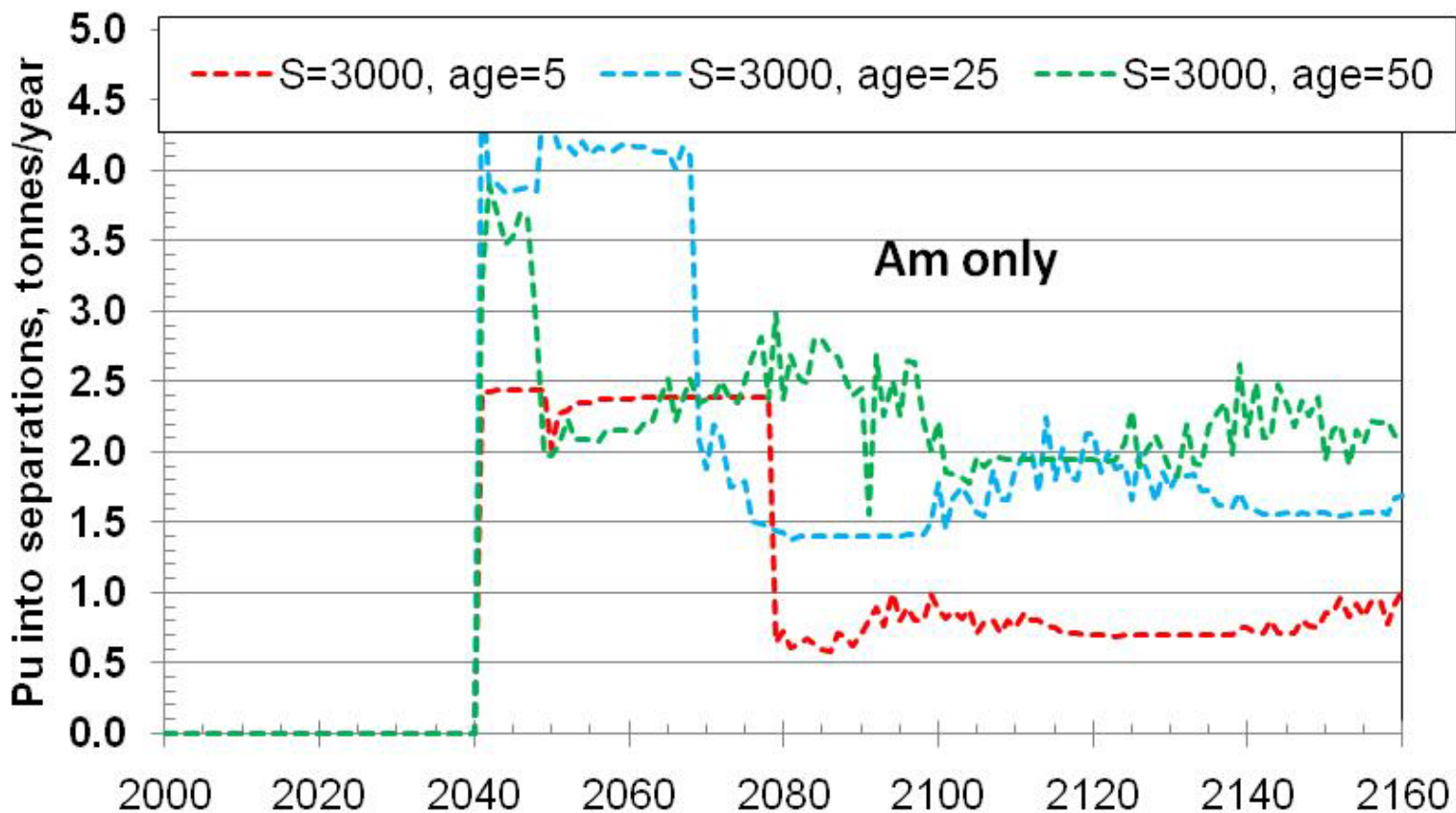
Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



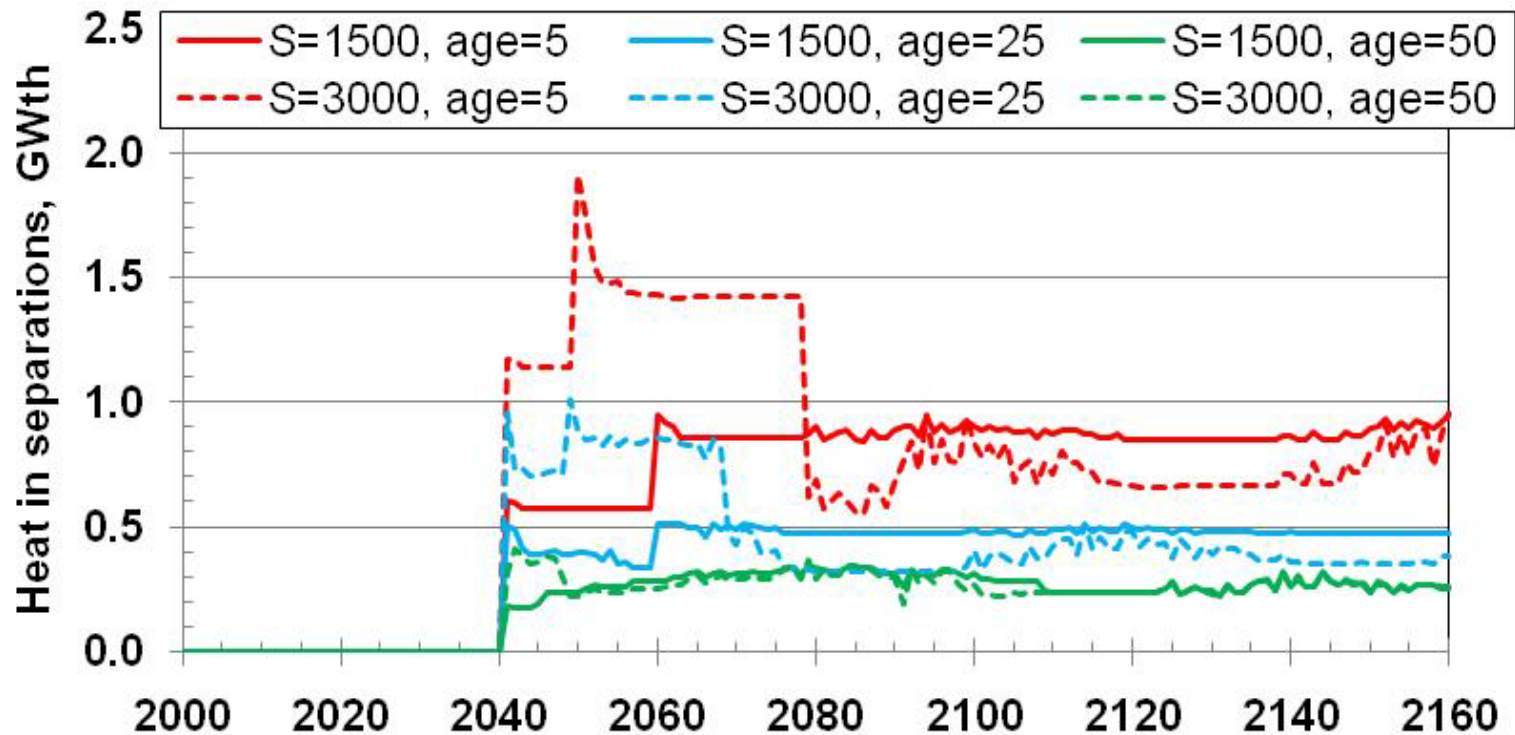
Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



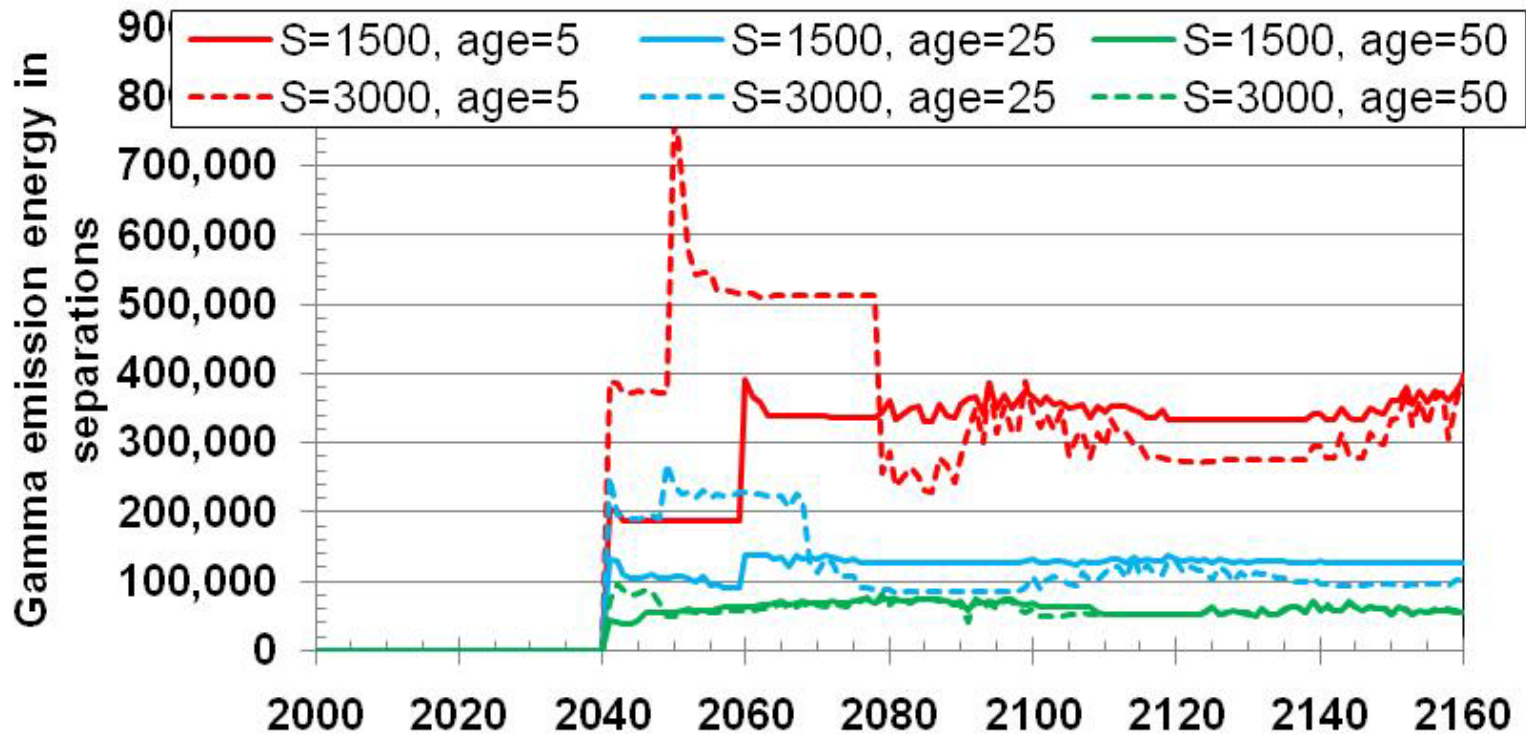
Phase 4 – Recycling rate (tonnes/yr) of PWR fuel



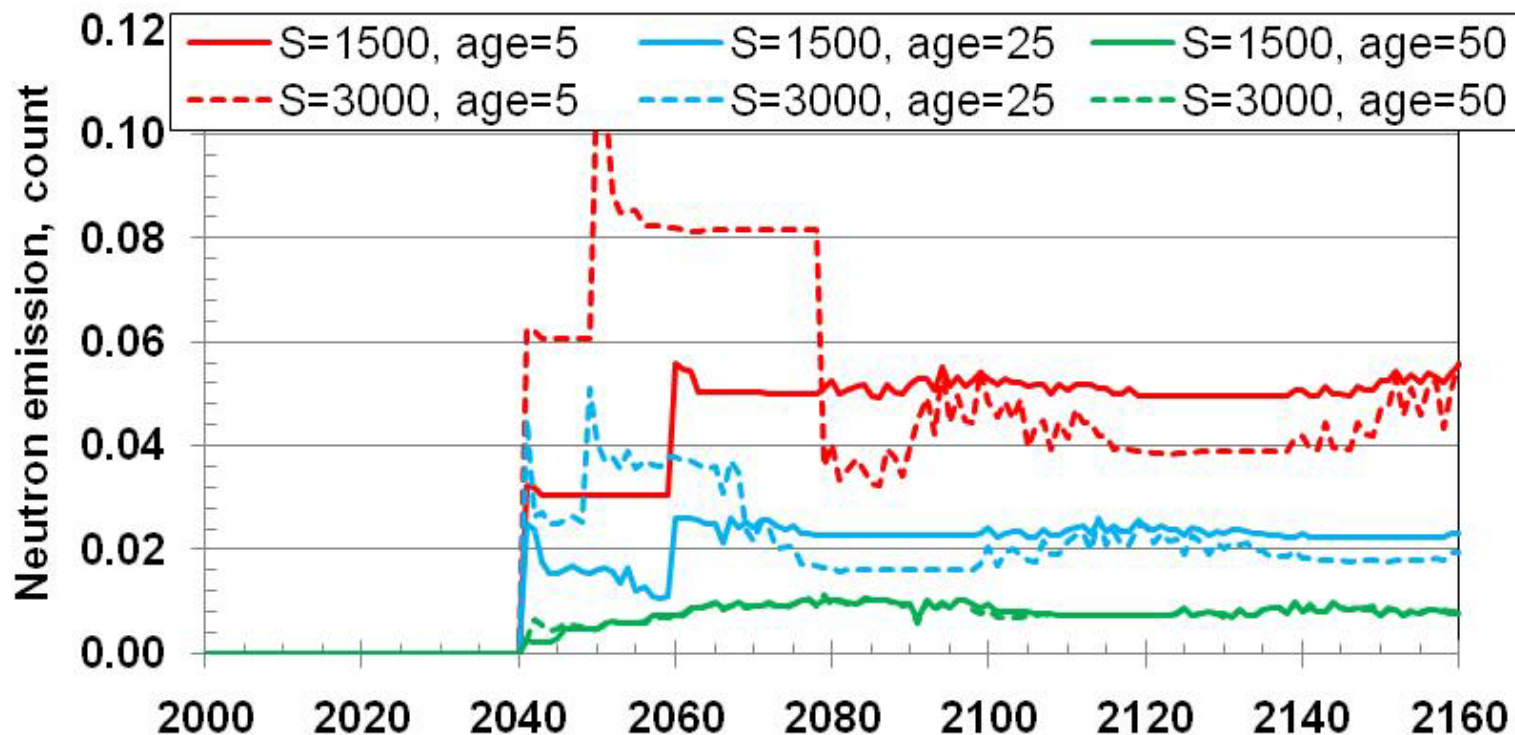
Phase 4 – Heat in separations (GWth) used fuel modeled as ¼ year in separations



Phase 4 – Gamma energy in separations used fuel modeled as 1/4 year in separations



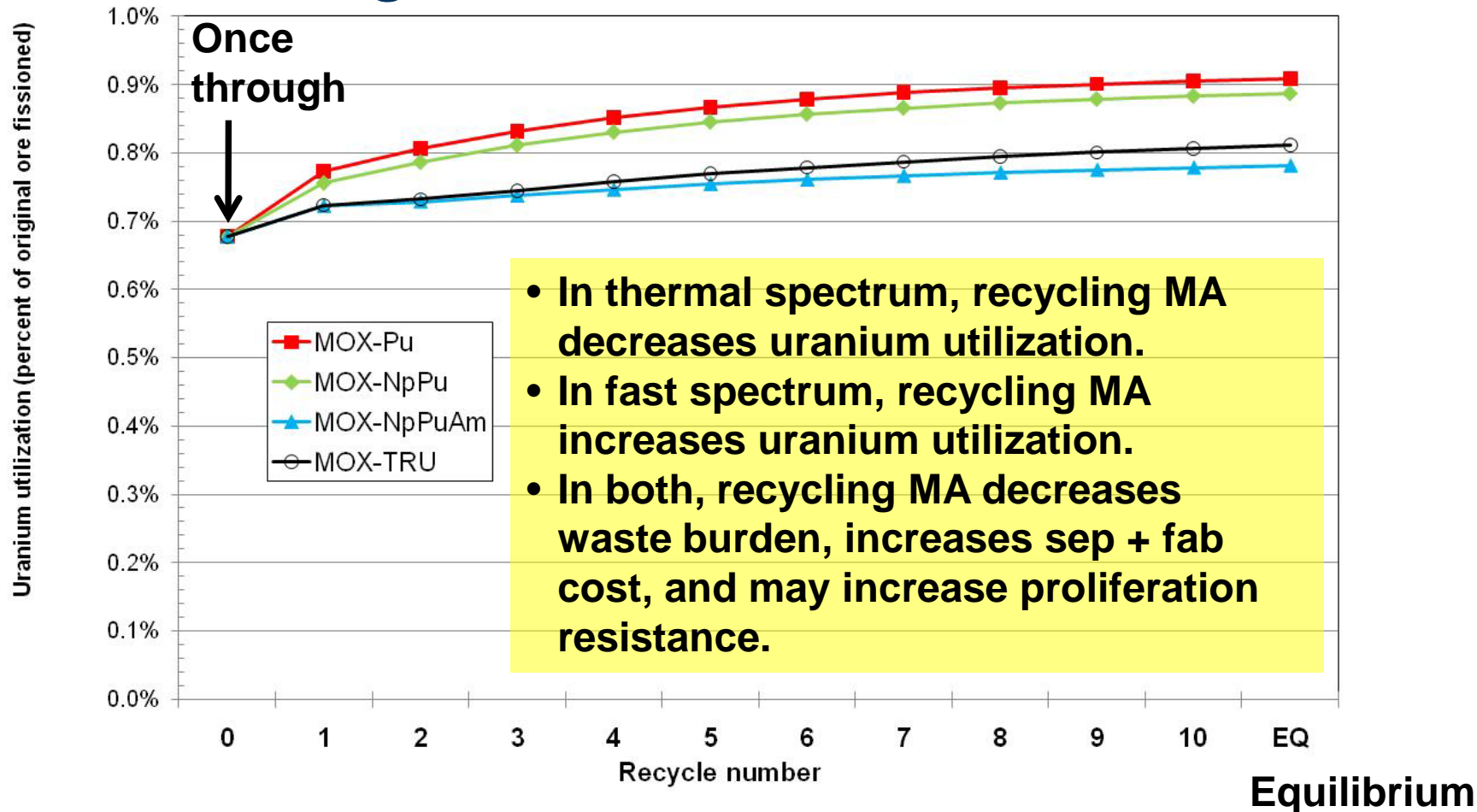
Phase 4 – Neutrons emitted in separations used fuel modeled as 1/4 year in separations



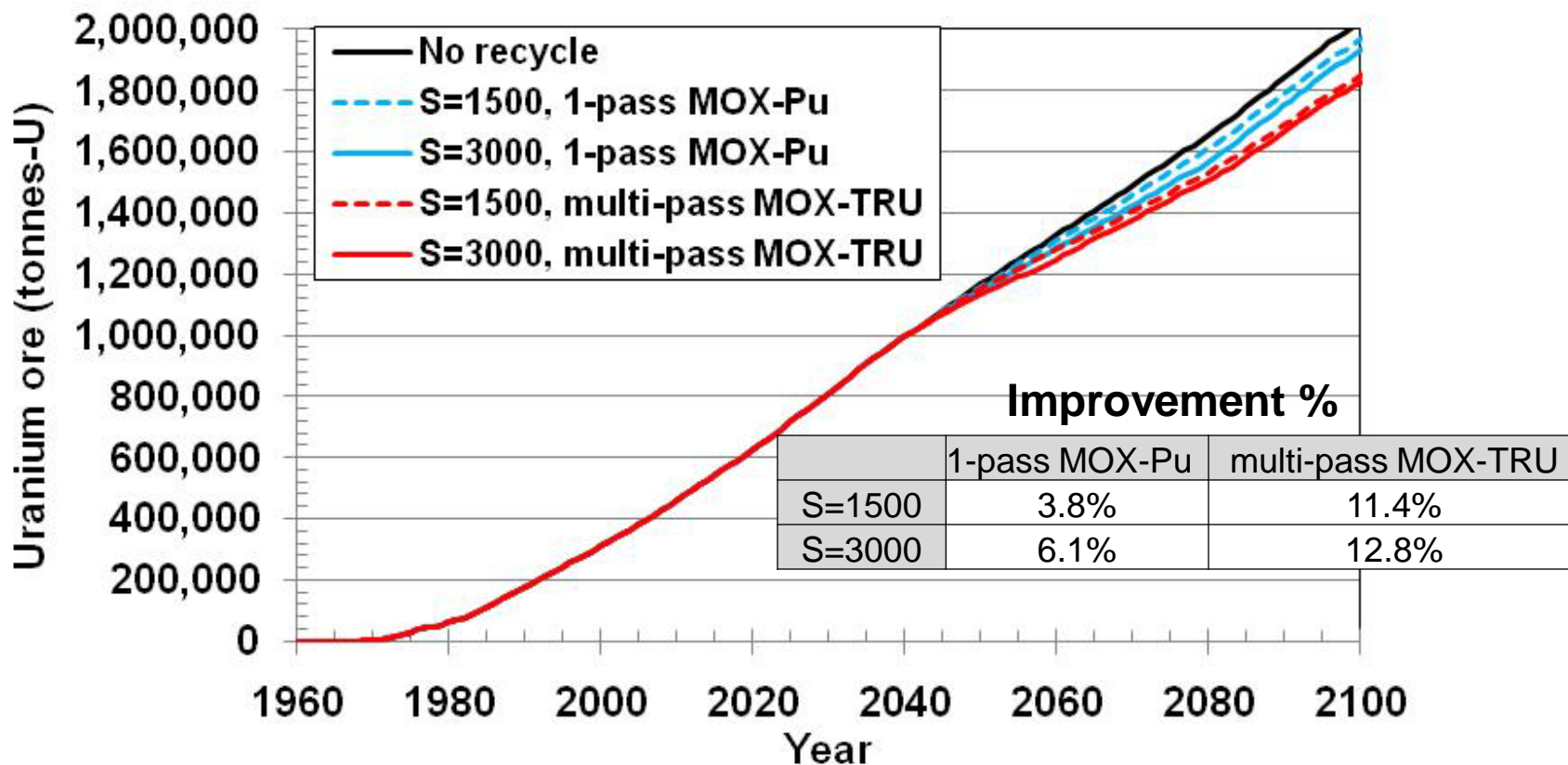
Phase 5 – Recycling + repository (no PWR UOX to repository)

- MOX-RU-Pu (1 recycle) – same as phase 4
 - Used MOX goes to repository
- MOX-EU-TRU (indefinite recycles), burnup = 51 GWth-day/tonne
 - Recycles TRU ($\leq 8\%$ of fresh MOX, enrich U as needed)
 - Used MOX always recycled, never to repository
- Repository opens in 2030 at 1500 tonnes/year, infinite capacity
- Separation cases specified, both start in 2040, tonnes-iHM
 - 1500 tonnes/year – backlog continues to grow
 - 3000 tonnes/year – backlog eliminated in 2115
- Minimum aging before separation = 5 years
- Per spec, BWR fuel not recycled
- Isotopic data in spreadsheet

Repeated LWR recycle possible, modest uranium savings

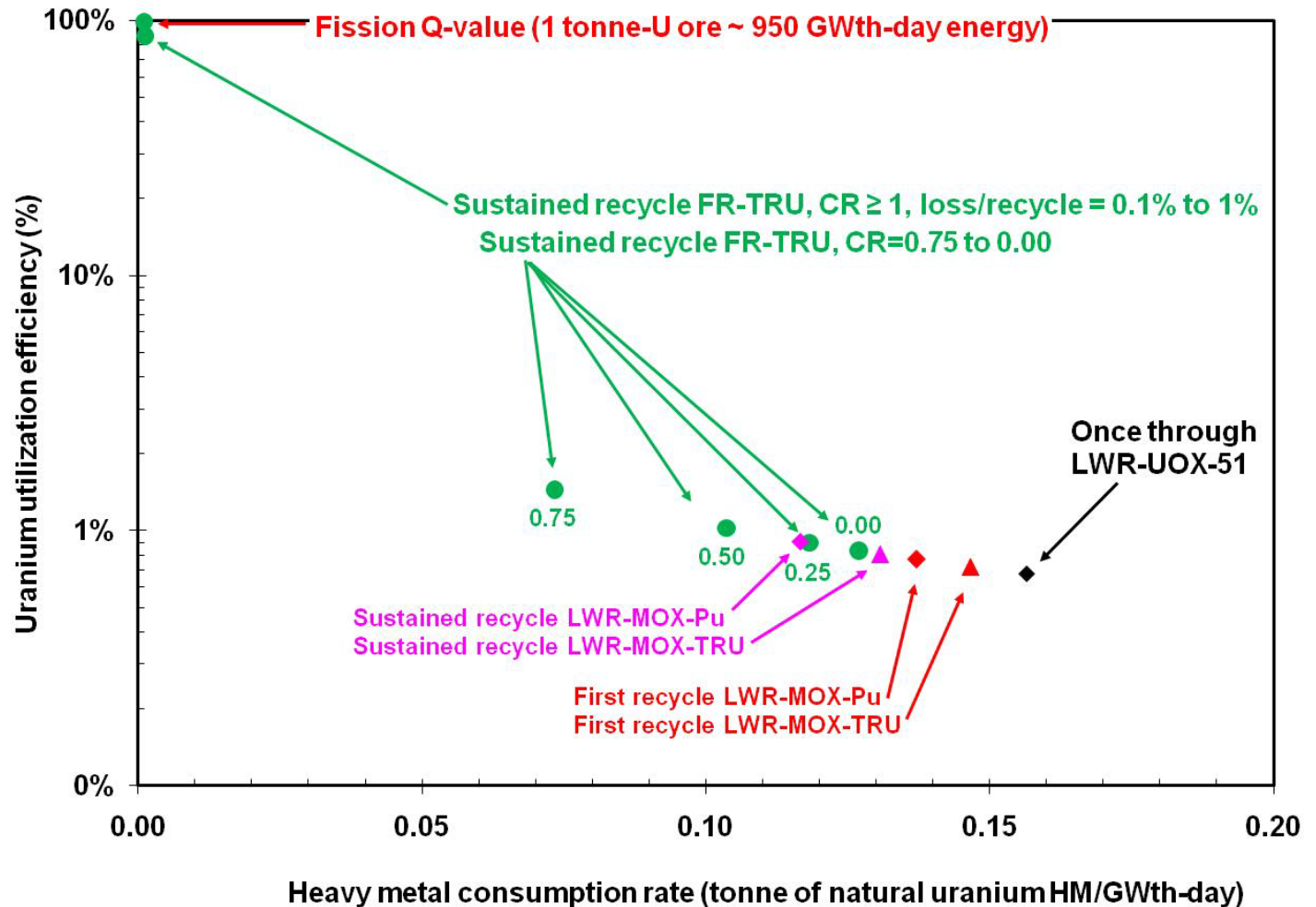


Phase 5 – Recycling – uranium ore



Phase 5 – Recycling

The broad picture on uranium



Phase 5 – Recycling – waste mass (1000s of tonnes, 2030-2100)

| PWR+BWR | Pu | MA | FP | U | Total |
|--------------------|-----|-----|------|-------|-------|
| No recycle | 2.2 | 0.4 | 10.7 | 196.7 | 210.0 |
| 1-pass MOX-Pu | 1.5 | 0.4 | 10.7 | 59.8 | 72.3 |
| Multi-pass MOX-TRU | 0.6 | 0.1 | 10.7 | 60.0 | 71.5 |

**Not necessarily high heat/
high longevity waste**

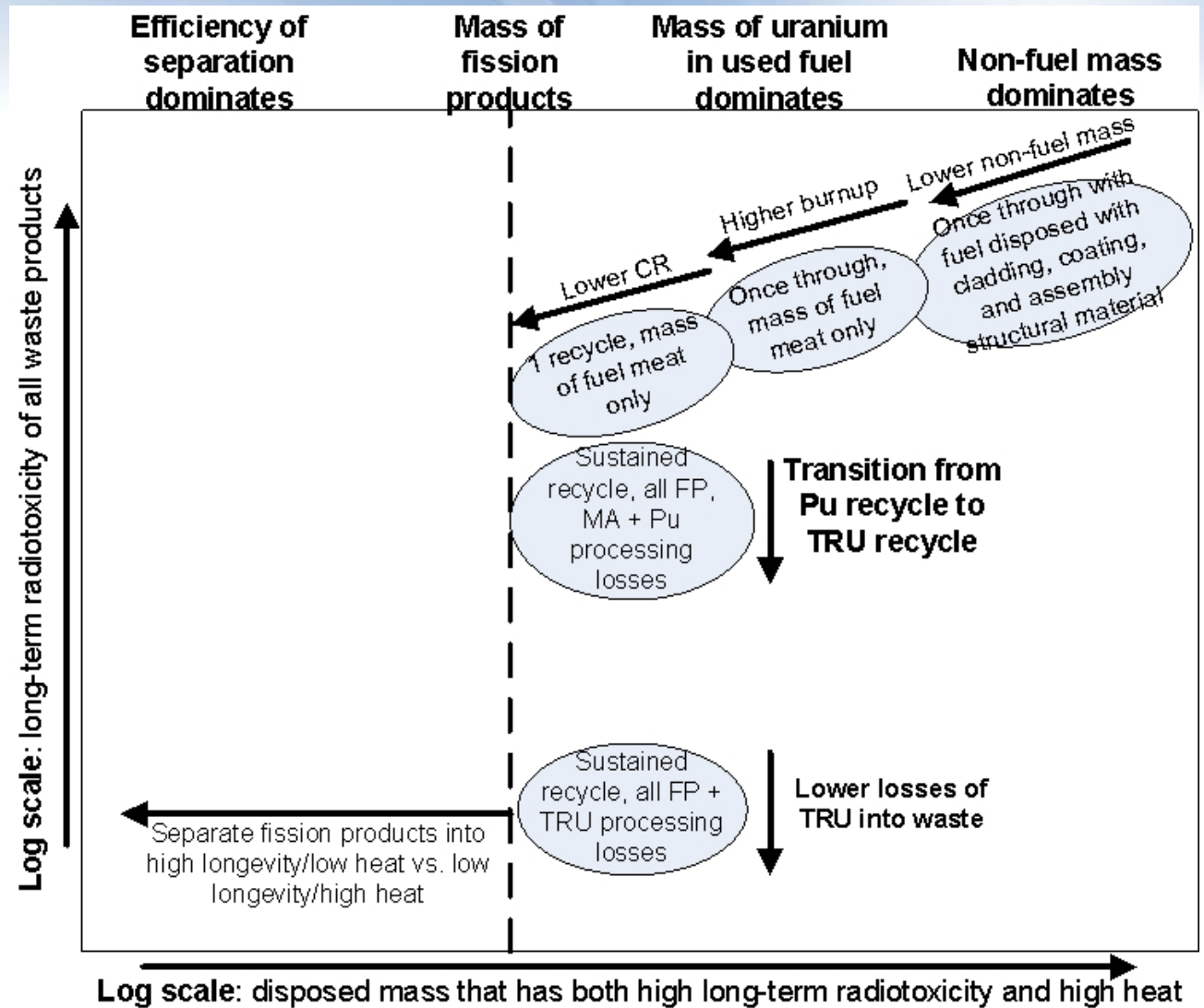


| PWR only | Pu | MA | FP | U | Total |
|--------------------|-----|-----|-----|-------|-------|
| No recycle | 1.5 | 0.3 | 7.4 | 133.6 | 142.7 |
| 1-pass MOX-Pu | 1.0 | 0.3 | 7.4 | 10.6 | 19.2 |
| Multi-pass MOX-TRU | 0.0 | 0.0 | 7.4 | 0.0 | 7.4 |

Could have sent RU from UOX separation as waste, but didn't.

Phase 5 – Recycling

The broad picture on waste



Conclusions

- Benchmarks and comparisons are tricky.
- Timing
 - Improvement of most metrics (uranium, waste, etc.) inhibited by delaying implementation and by increasing UNF cooling time (hence increasing time lag in recycling)
 - Improvement inhibited if only recycle PWR (not BWR)
 - Of course, more recycling may increase cost.
- Uranium use
 - MOX achieves minor uranium savings
 - Higher if multi-recycle
 - If uranium savings is the goal, use a breeder reactor
- Waste management
 - If waste mgt savings is the goal, keep recycling