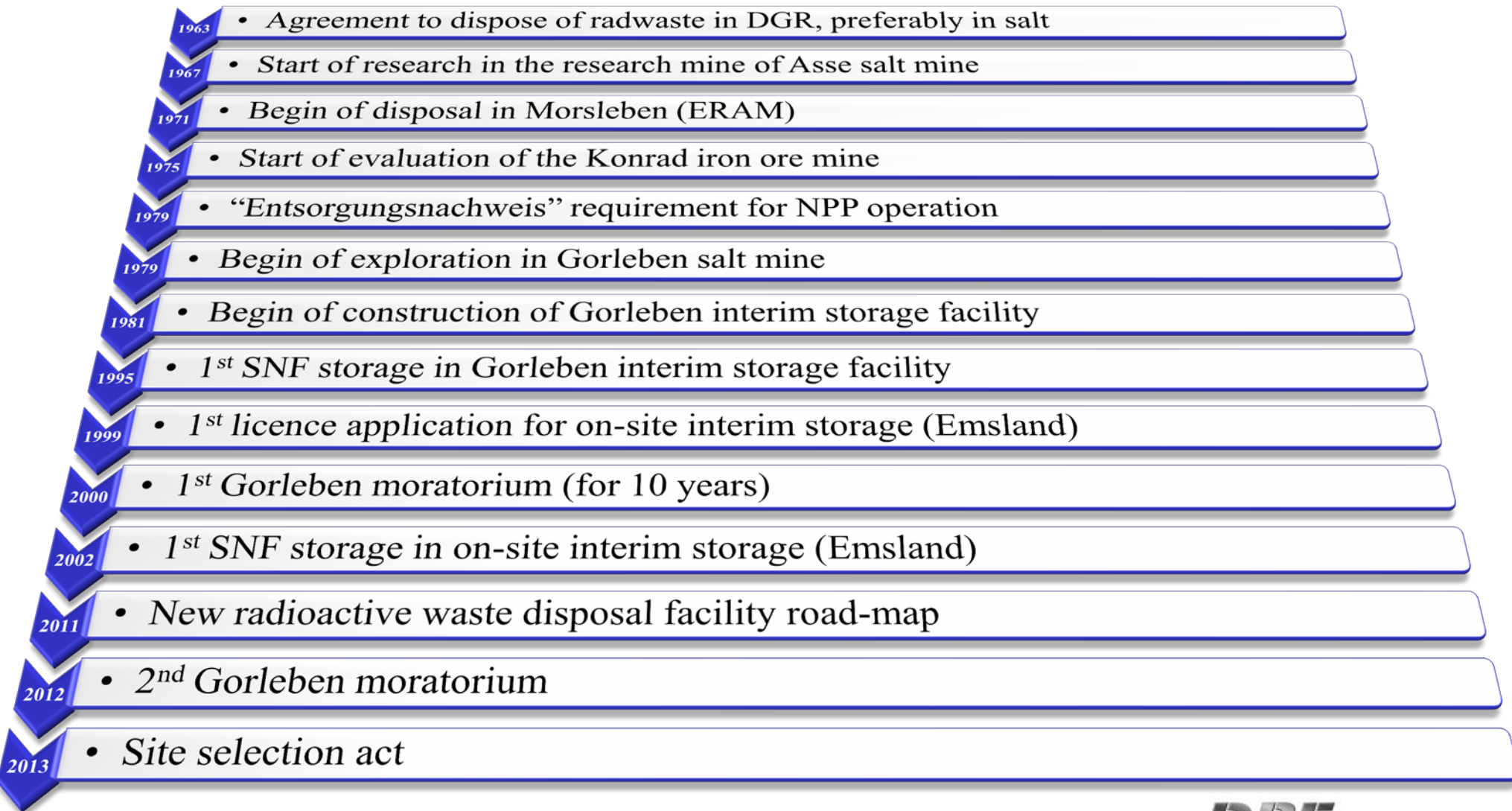

International Perspectives on the Impacts of the Design of the Spent Fuel Management Programme on Spent Fuel Handling, Transportation and Disposal

Dr. Thilo v. Berlepsch
DBE TECHNOLOGY GmbH

== Milestones of Radioactive Waste Management in Germany =



Some Assumptions and Definitions ...

Borehole Emplacement

- NOT deep boreholes, rather 'tight drilled vertical drifts'

BSK3

- Disposal only casks for borehole emplacement

CASTOR®

- Transport and storage cask

Conditioning

- Here: Preparation for disposal – Originally planned at the repository site (nuclear waste management center) with the full scale pilot plant still near the Gorleben exploration mine, but principally without formal decision on the final location

CSD-C

- Compacted waste packages from reprocessing in France

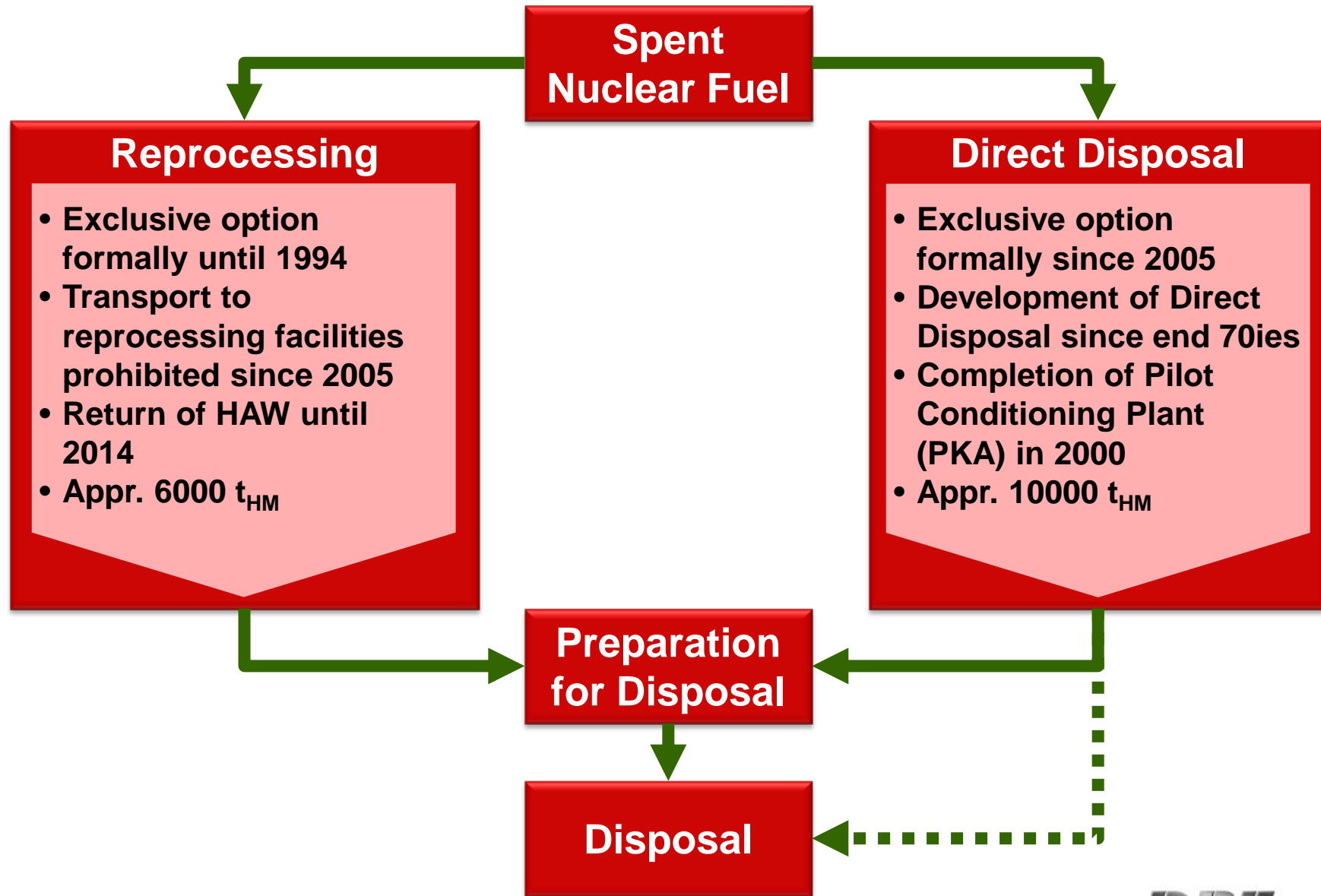
DIREGT

- Concept for the direct disposal of CASTOR® casks

POLLUX®

- Transport and disposal cask

Spent Nuclear Fuel Management in Germany



== Main Concepts for Spent Nuclear Fuel Disposal ==

Reference Concept

Additional Concepts

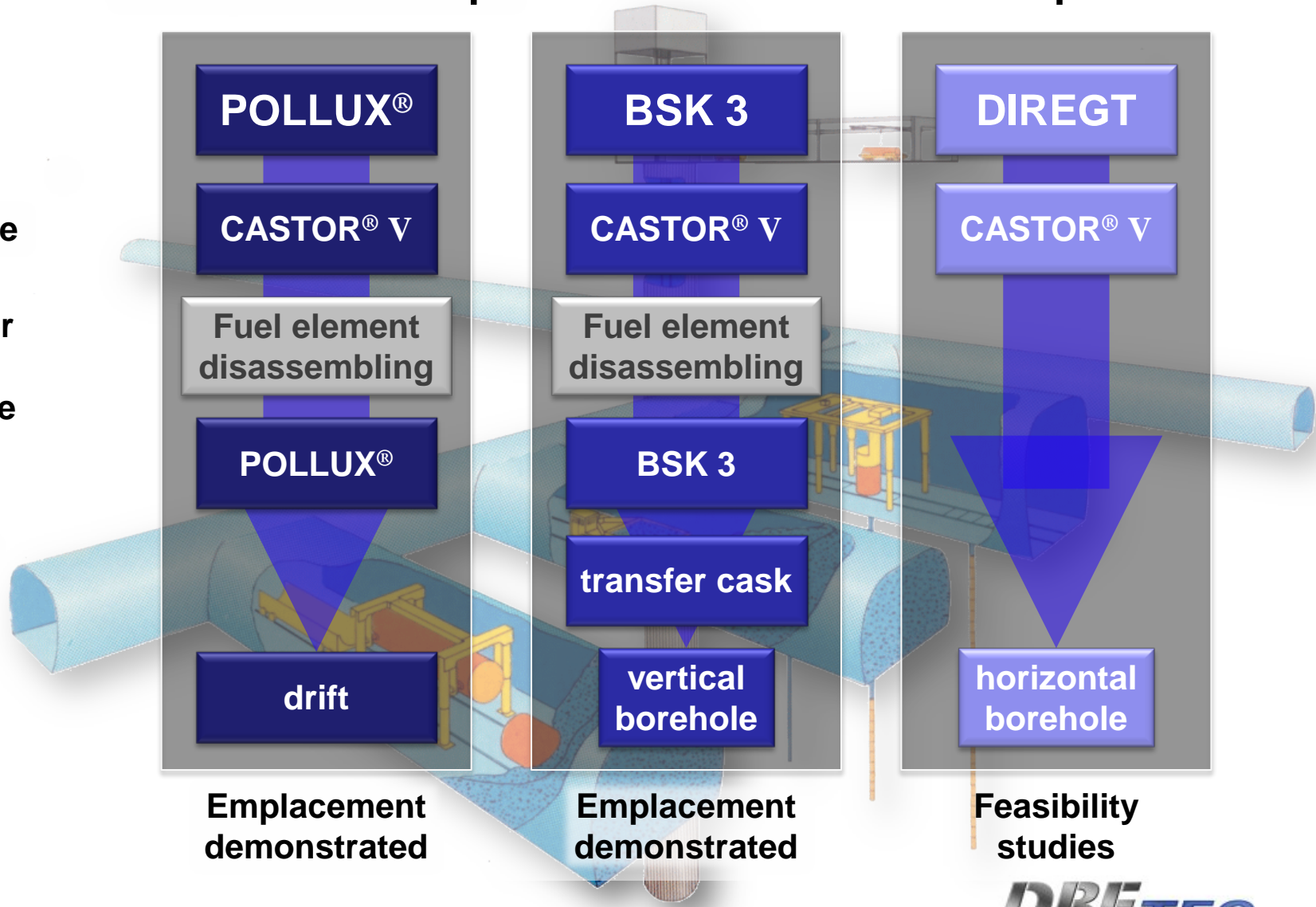
On site storage

Preparation for Disposal after interim storage

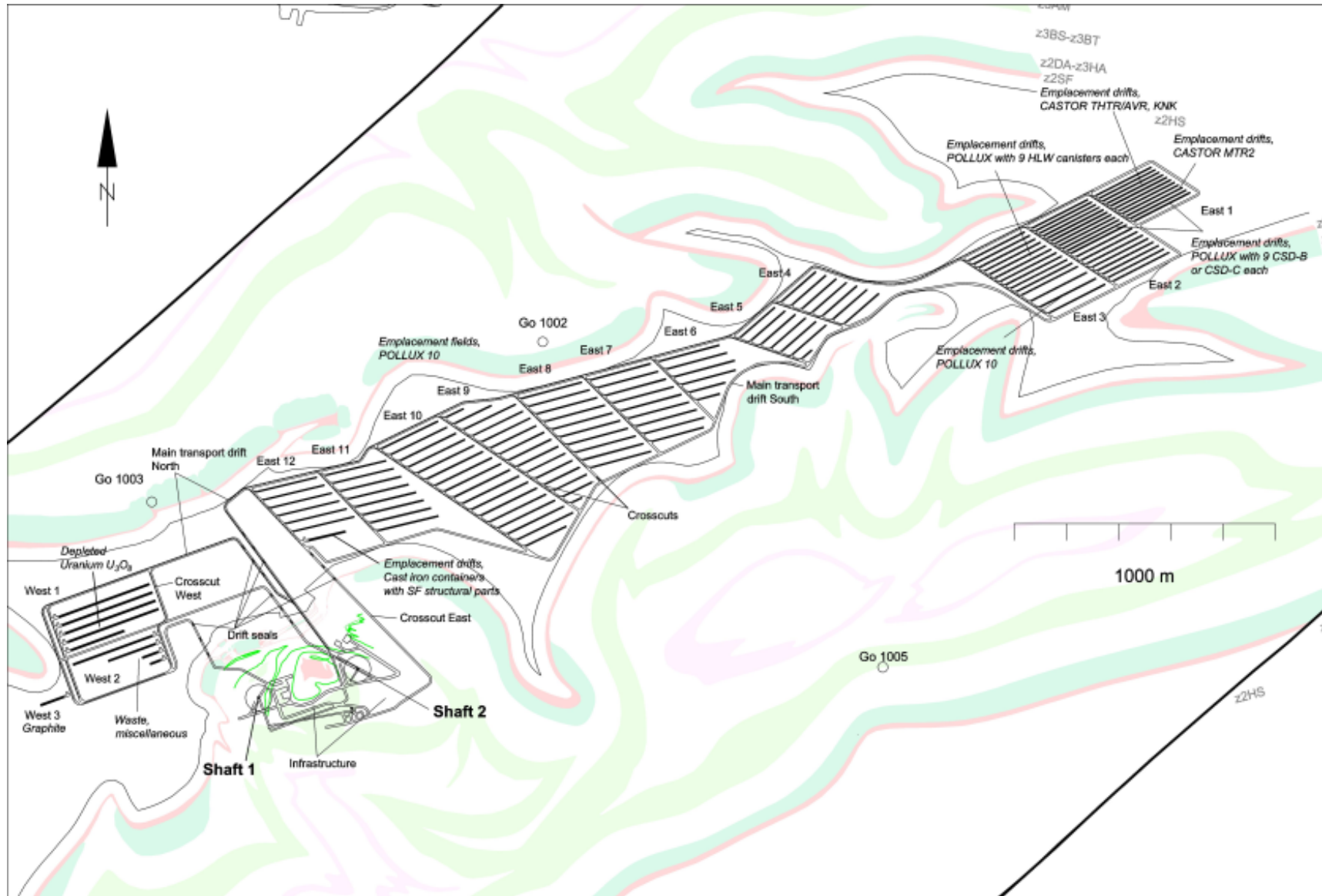
Transfer to final disposal

Final disposal

Status

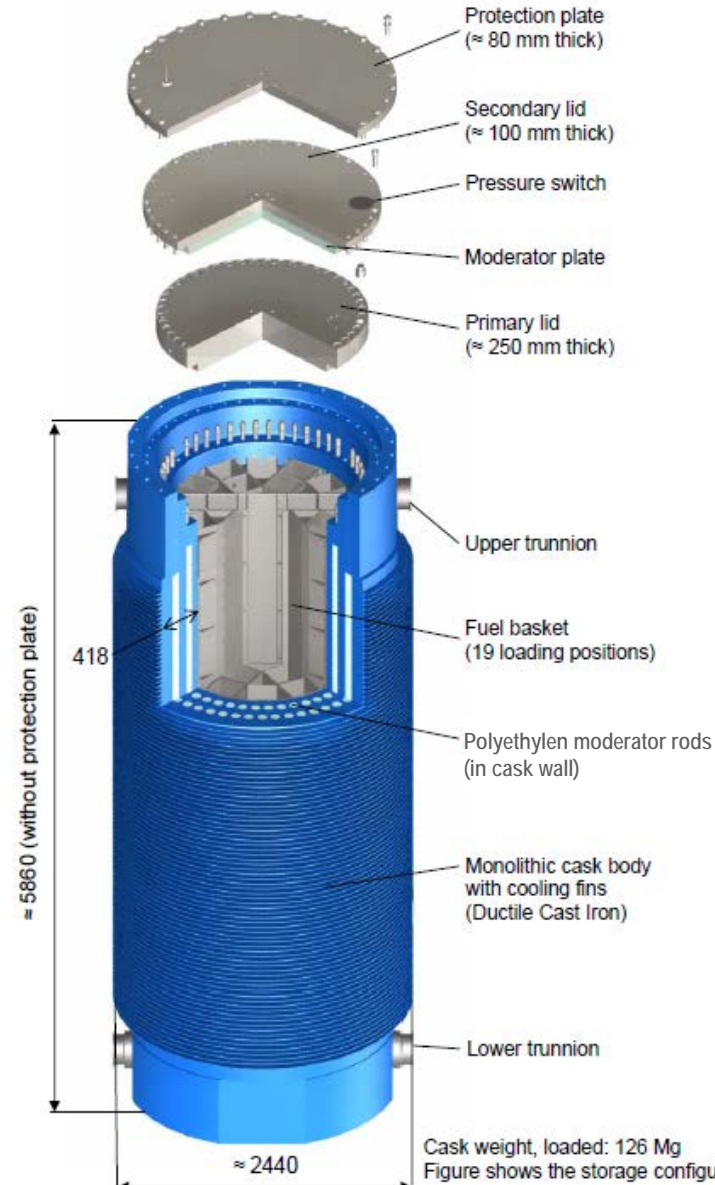


Reference Concept – Repository Design



Reference Concept – On Site Storage

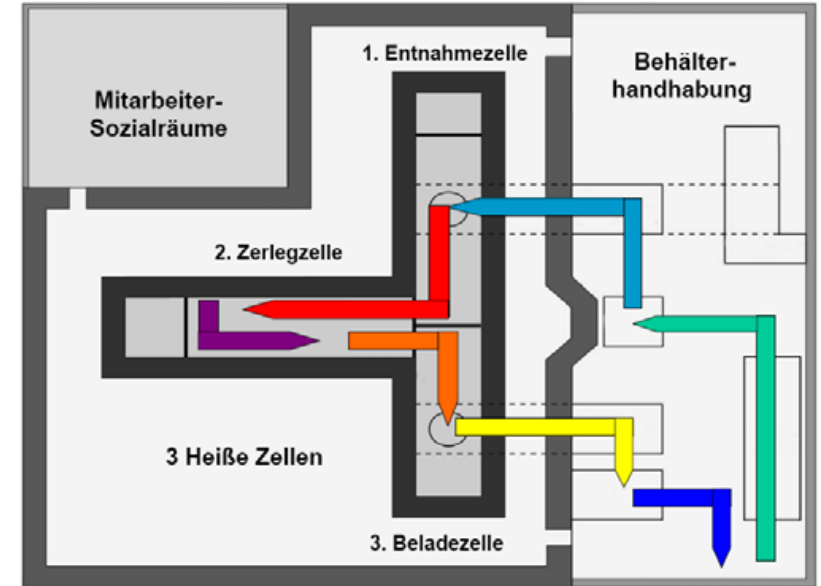
CASTOR[®] V/19



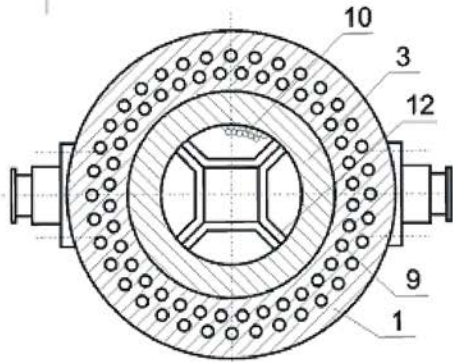
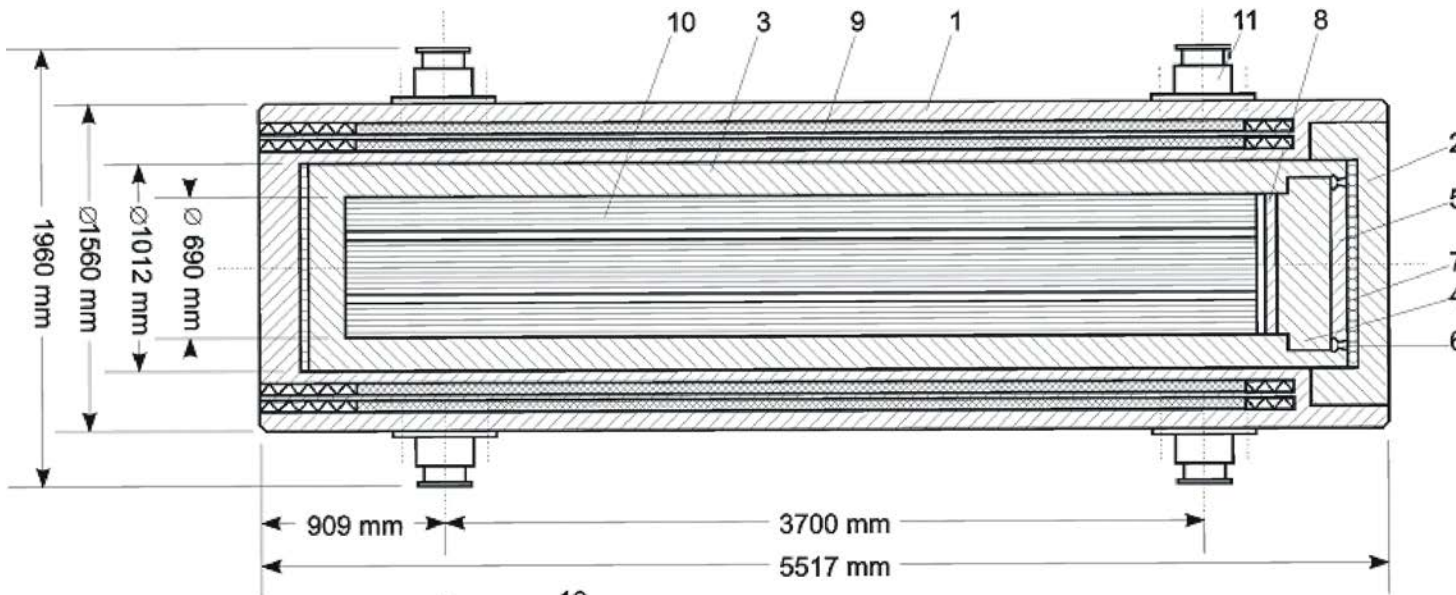
**125 t CASTOR[®] cask
for max. 19 PWR spent
fuel elements**

(Source: GNS)

Reference Concept – Preparation for Disposal



Reference Concept – Preparation for Disposal



- 1 Shielding Cask
- 2 Shielding Lid
- 3 Inner Cask
- 4 Primary Lid
- 5 Welded Secondary Lid
- 6 Welding Seam
- 7 Damping Element
- 8 Moderator Plate (Graphit)
- 9 Moderator Rods
- 10 Fuel Rods
- 11 Trunnion
- 12 Basket Structure

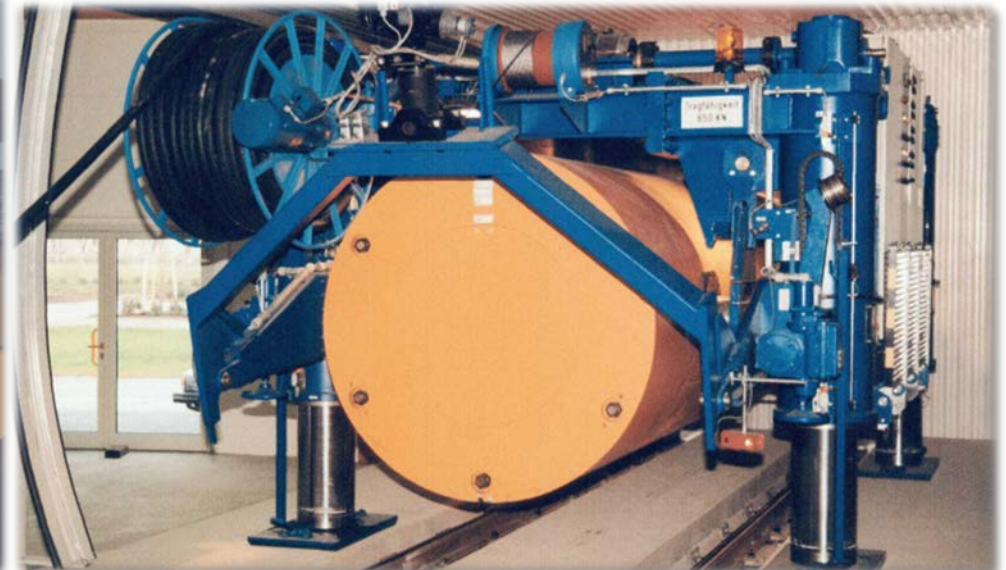
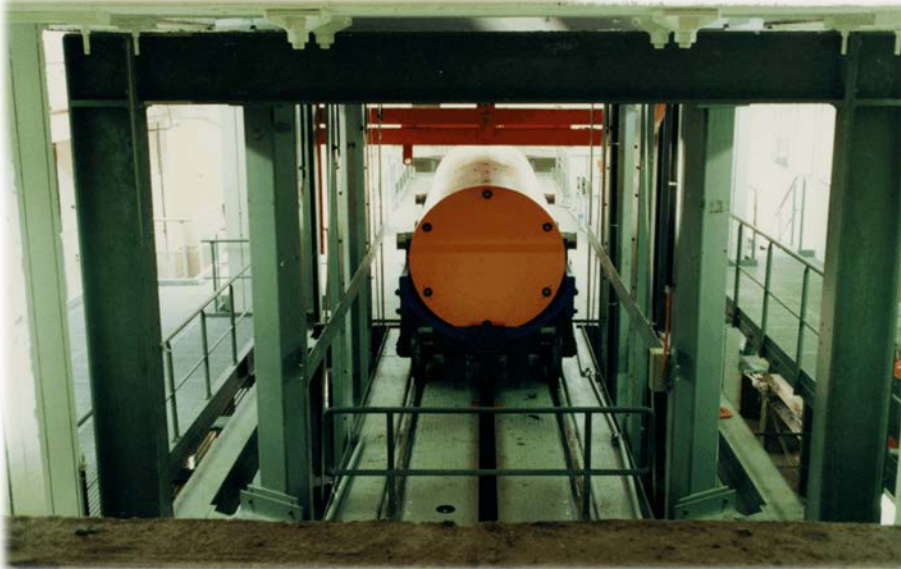


(Source: GNS)

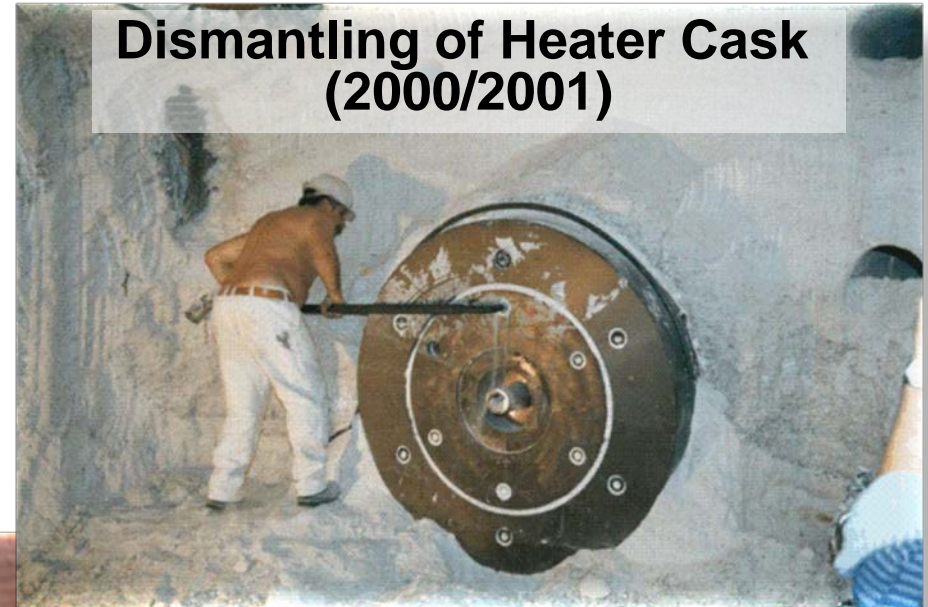
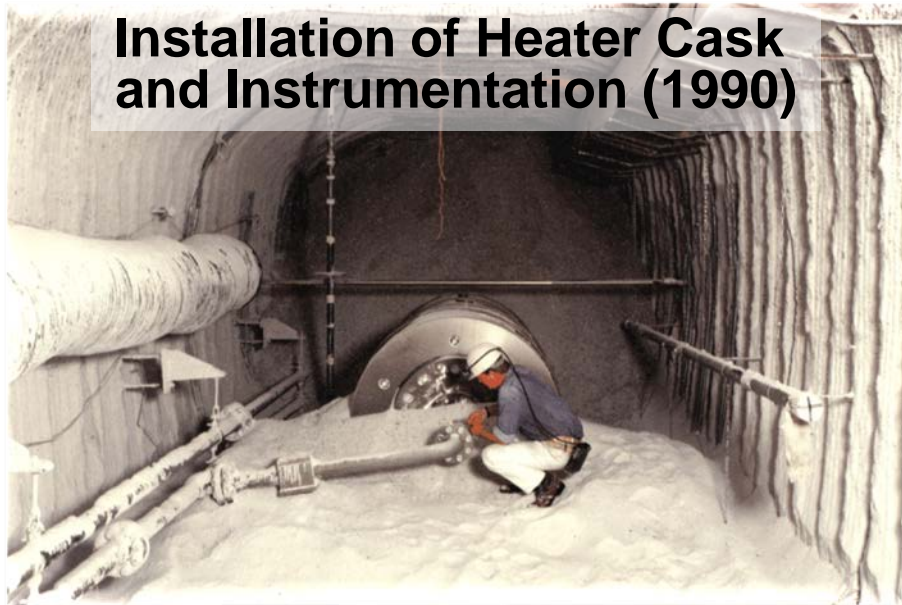
DBETEC
DBE TECHNOLOGY GmbH

**65 t POLLUX[®] cask
for max. 10 PWR spent
fuel elements**

Reference Concept – Transfer to Final Disposal



Reference Concept – Final Disposal



BSK Concept – Motivation

Emplacement technologies for both categories of waste (vitrified waste and spent fuel)

Emplacement technology which can be used for the handling and emplacement of different types of casks

Improvement of heat transfer from waste canister to the host rock (rock salt) by means of close contact

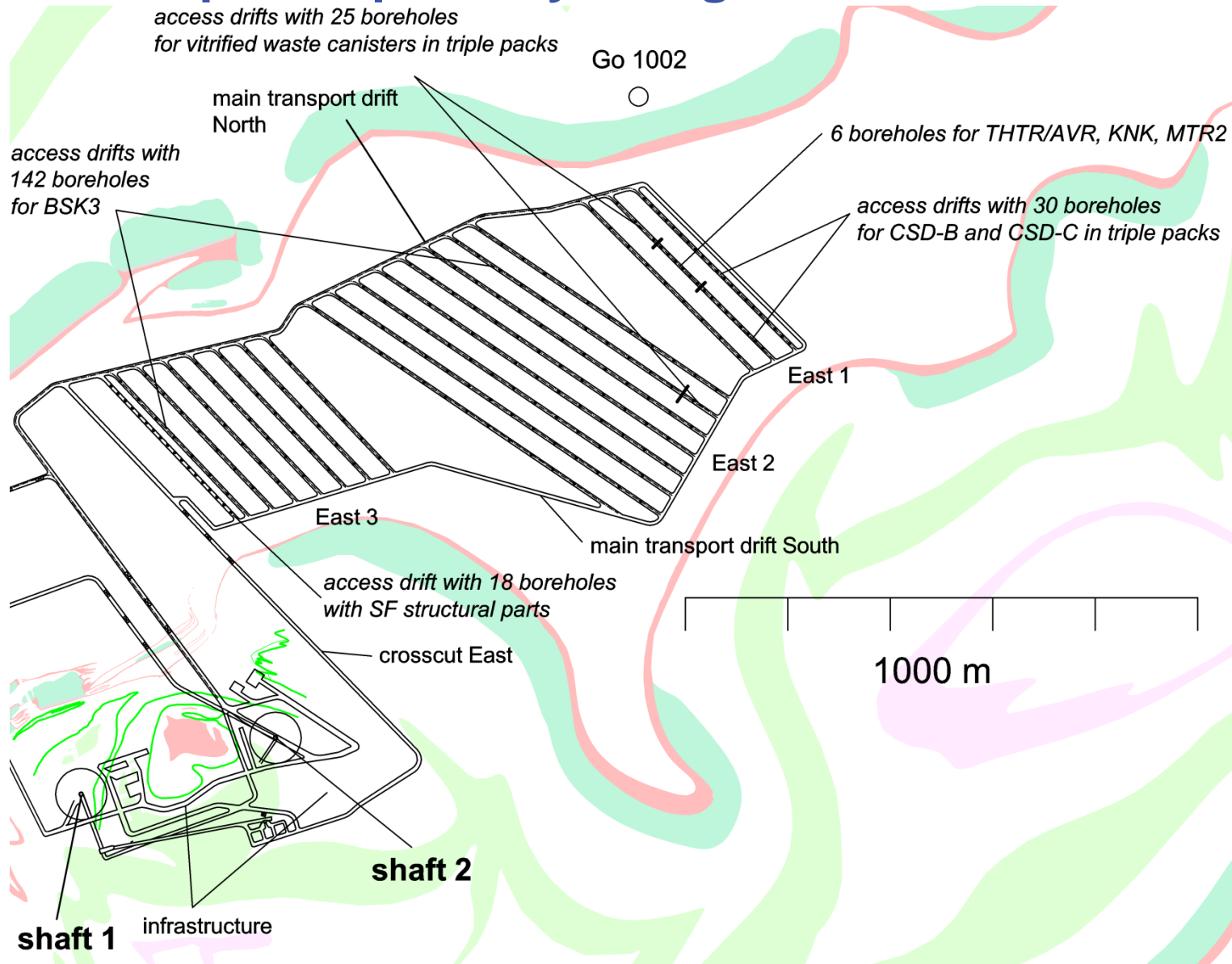
Compared to the POLLUX emplacement concept a faster process (creeping of host rock) to achieve the entire enclosure of the waste cask by host rock

Reduction of required footprint of the repository by using the host rock in three dimensions

Economical benefit by saving cask and operating costs

Reduction of potential gas generation (corrosion) due to reduction of metallic material mass

BSK Concept – Repository Design



BSK Concept – Preparation for Disposal

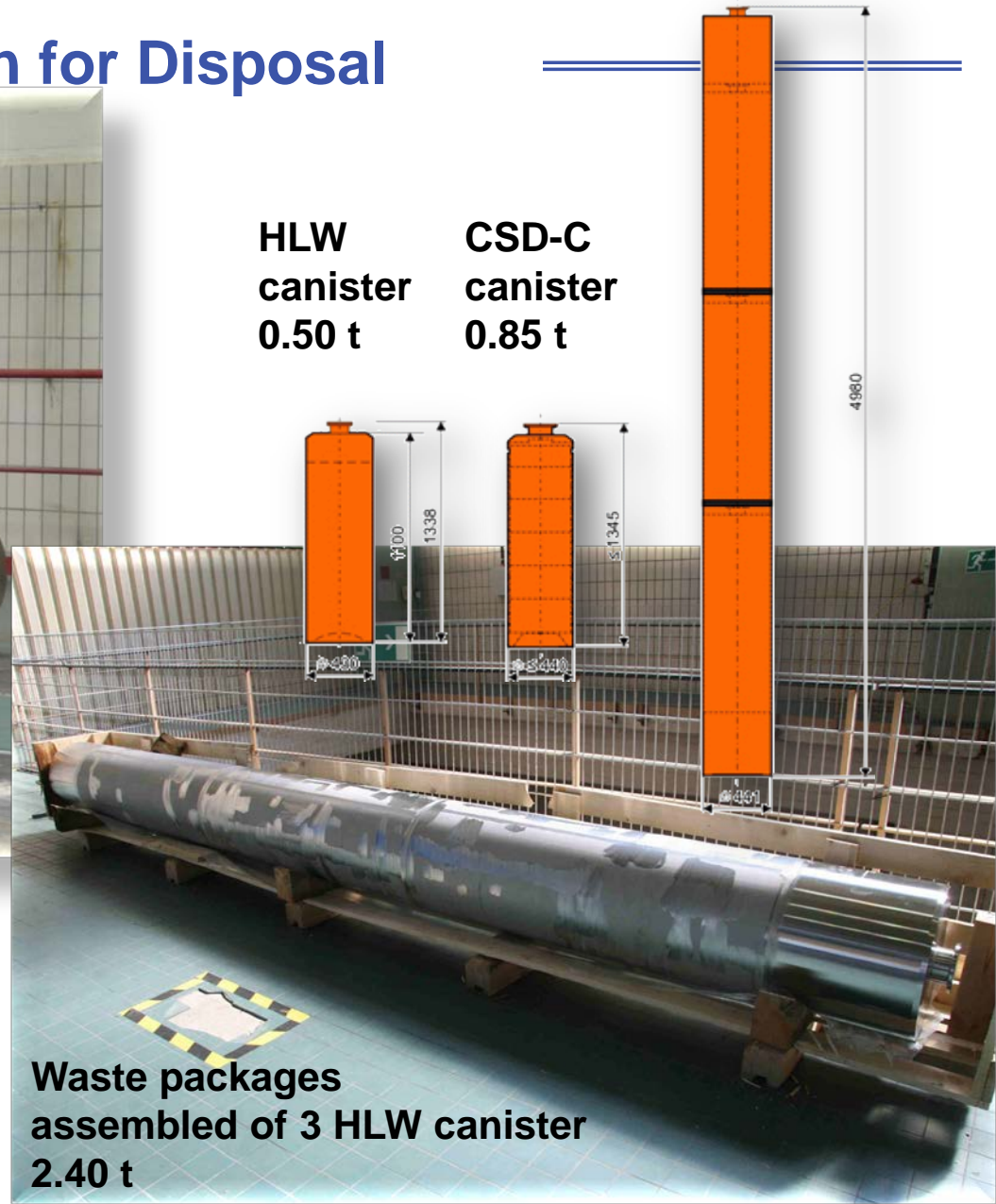


**BSK 3
canister
5.20 t**

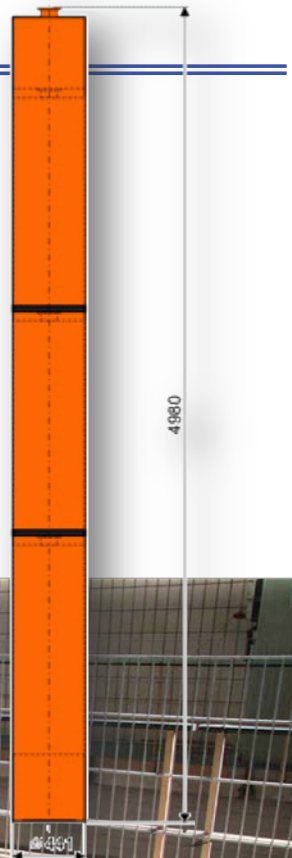
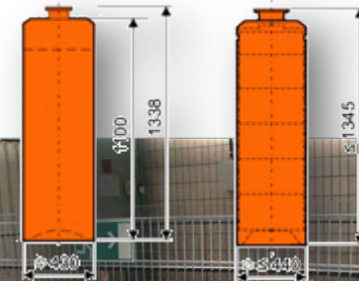


**HLW
canister
0.50 t**

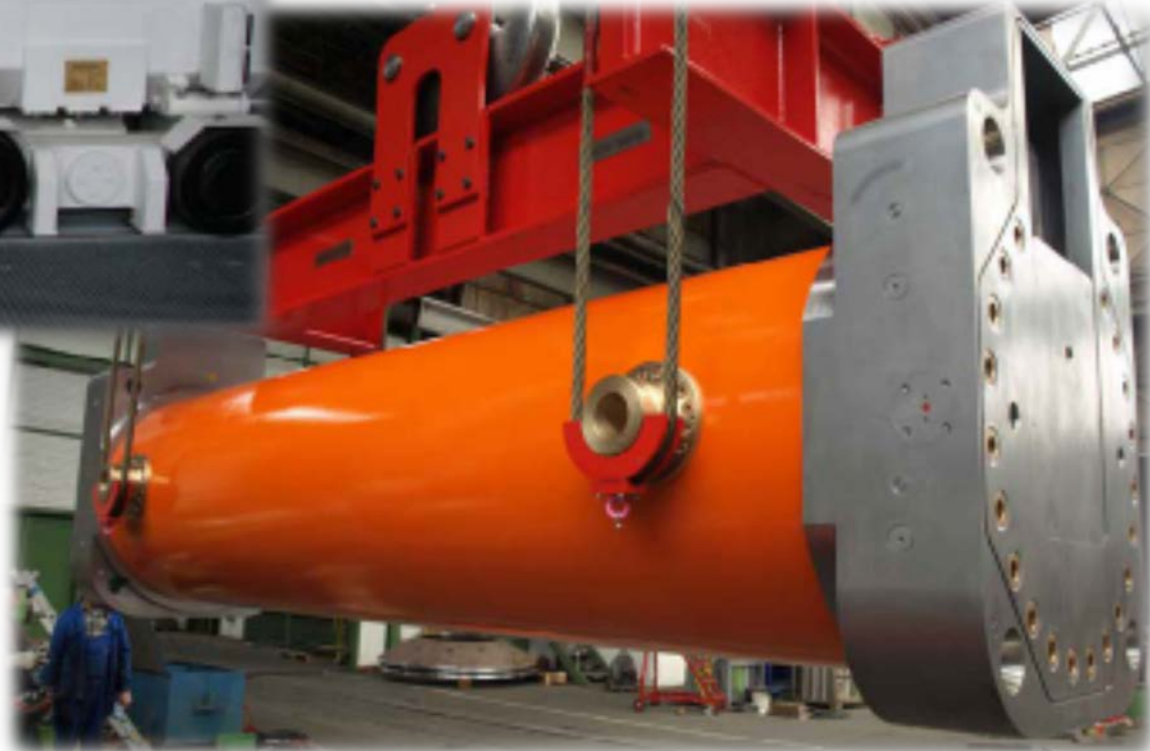
**CSD-C
canister
0.85 t**



**Waste packages
assembled of 3 HLW canister
2.40 t**



== BSK Concept – Transfer to Final Disposal ==



BSK Concept – Final Disposal



== DIREGT – Motivation ==

Consistent emplacement technology for direct disposal of nuclear waste from reprocessing and spent nuclear fuel

Emplacement technology for disposal in transport and storage casks (CASTOR)

Avoidance of Preparation for Disposal / encapsulation of spent nuclear fuel

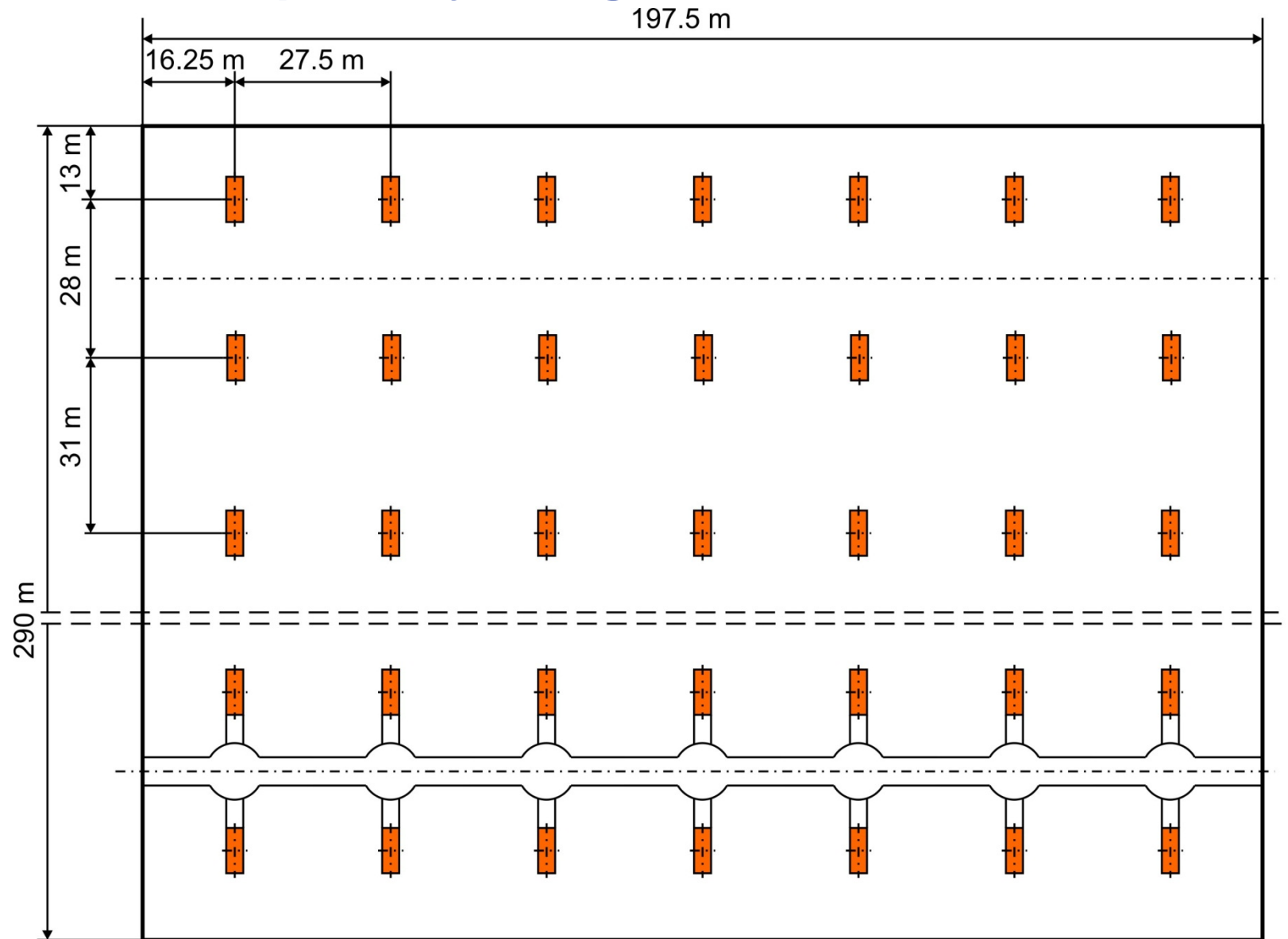
Prevention of time consuming and complex processes requiring severe radiation protection

Minimising the transports of casks to the repository

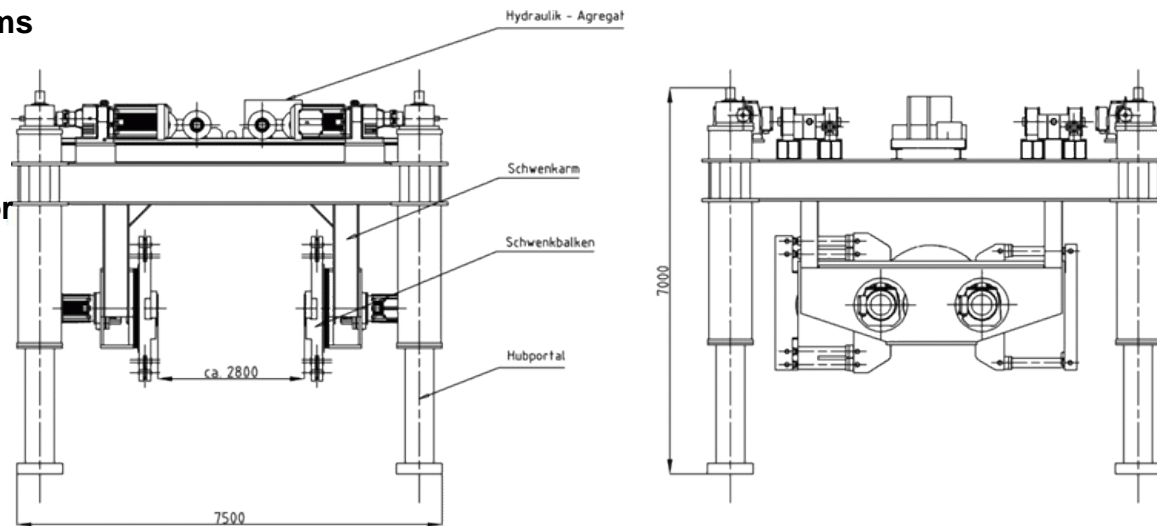
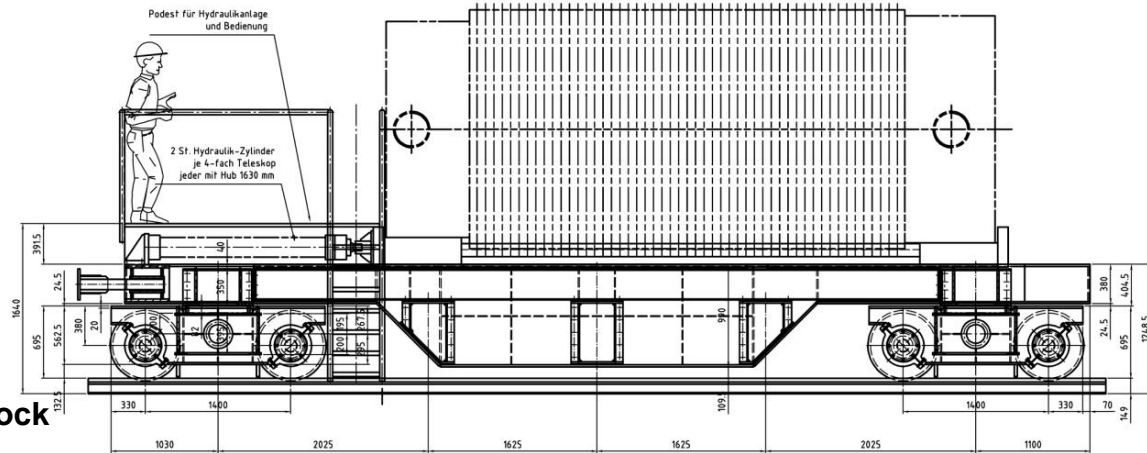
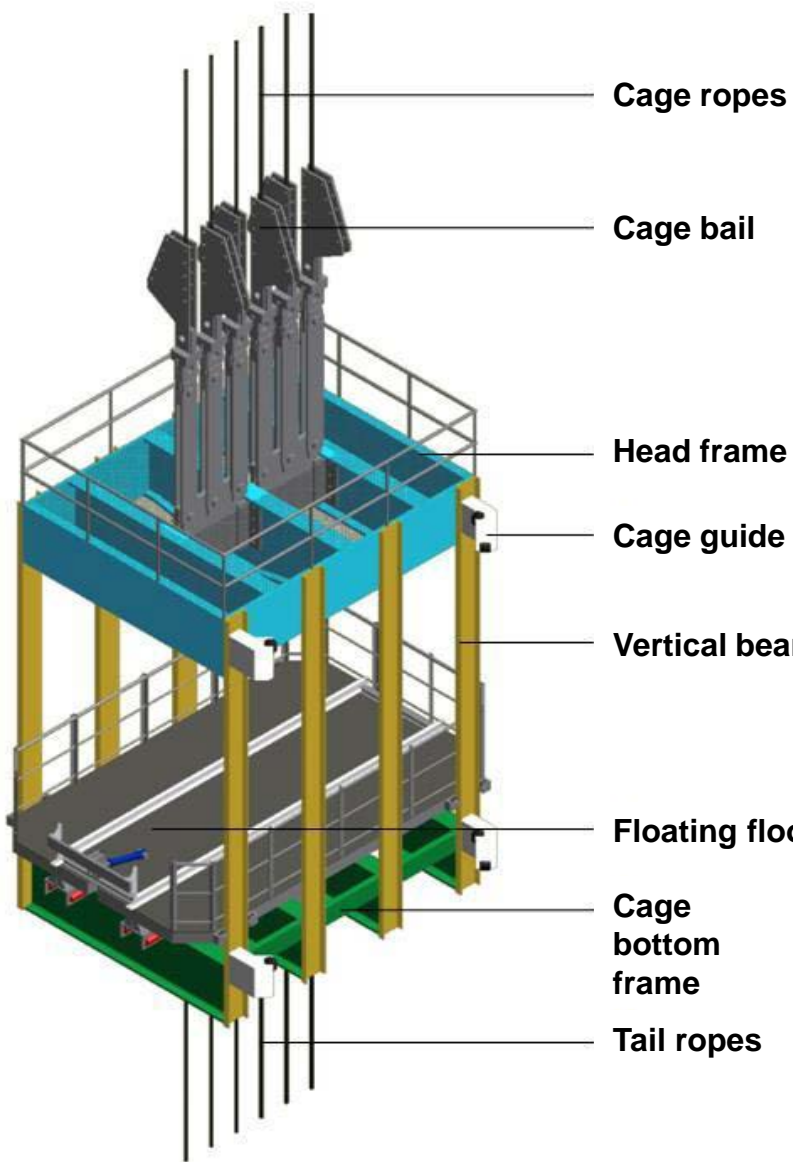
Minimising general handling activities with casks

Concurrent disposal of redundant transport and storage casks

DIREGT – Repository Design

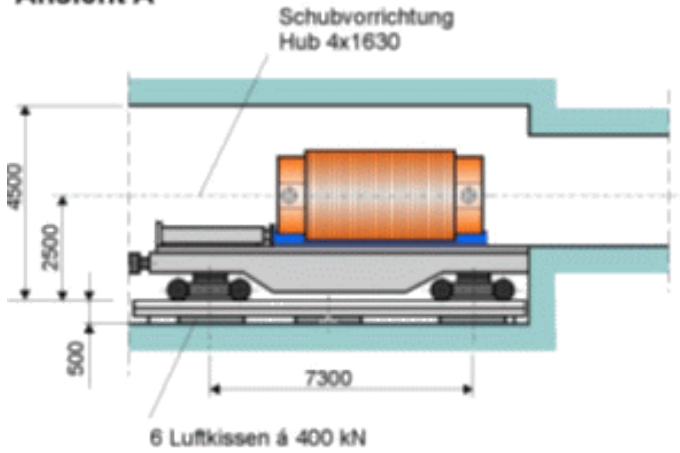


DIREGT – Transfer to Final Disposal

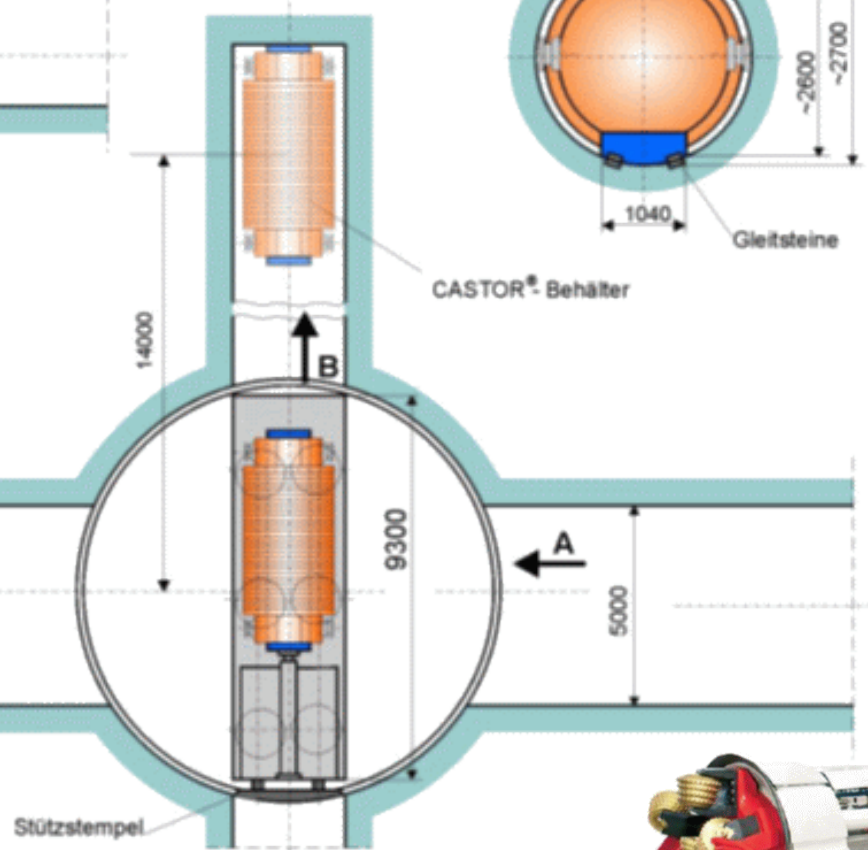
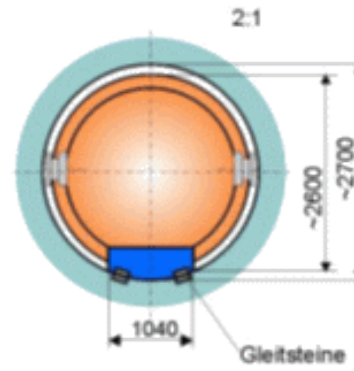


DIREGT – Final Disposal

Ansicht A



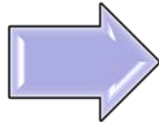
Ansicht B



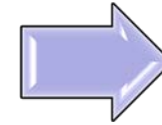
(Source: Sandvik)

== Major Limitations of BSK and DIREGT concept ==

Reference
Concept



BSK Concept



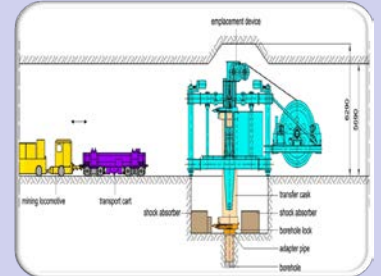
DIREGT

- Feasible for deep layers only
- Demonstration of retrievability
- Proof of backfilling
- Low radiation shielding requires complex emplacement technology
- High radiation impact on host rock

- Handling of very heavy loads
- Voids in casks
- Criticality
- Up to now only concept, no demonstration yet

Challenges

Challenges generally encounter when commencing with detailed planning of operation or even when starting operation, e. g.:



Challenging handling equipment design due to different cask designs

Specific requirements on hoisting system due to transport of very heavy loads

Uncommon radiation protection issues due to back-scattering of neutrons by the host rock

Complex and time consuming activities due to limited space underground

To ensure cost effectiveness excavations have to be as little as possible