

Technical Basis for Disposal of Spent Nuclear Fuel and High-Level Radioactive Waste in Salt

Kristopher L. Kuhlman Sandia National Laboratories

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Technical Basis for HLW in Salt

What is a "Technical Basis"?

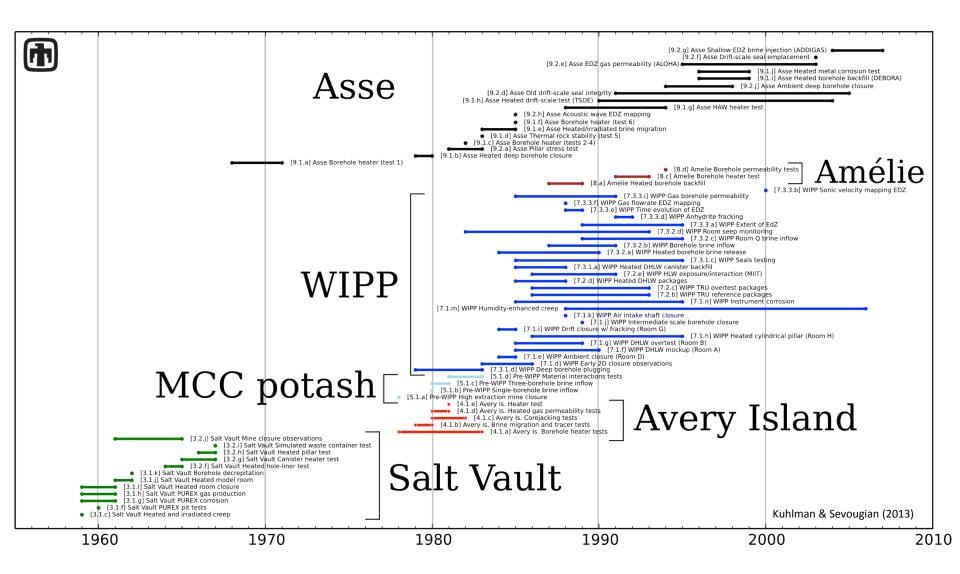
- Achieved through iterative process:
- 1. Understand Relevant Processes
- 2. Develop Conceptual/Mathematical/Numerical Models
- 3. Parameterize/Validate Models with Observations
- 4. Quantify Limitations and Uncertainty in Models

Outline:

- Highlights of HLW-related testing in salt
- What has been learned?
- What remains?



Heated Salt In Situ Testing Timeline





U Texas Lab Testing

NAS panel

Nuclear Energy

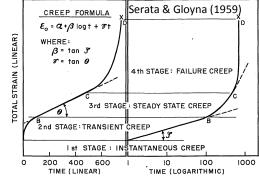
 disposal of liquid reprocessing waste in salt domes (Hess et al., 1957)

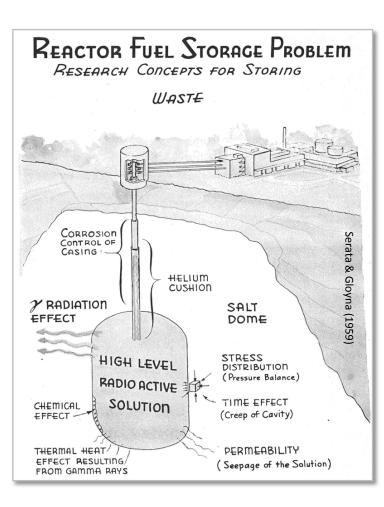
U Texas Austin performed early lab testing

- Uniaxial creep (≤ 410° C)
- Cavity closure
- Salt permeability (k) testing
 - He, brine, and kerosene flow
 - Crystals are impermeable
- Closure observations Grand Saline Mine (Dallas, TX)

Learned:

 Early geomechanical tests validated thermomechanical theory







ORNL Pre Salt Vault: Hutchinson, KS

Nuclear Energy

Heated liquid PUREX waste in salt

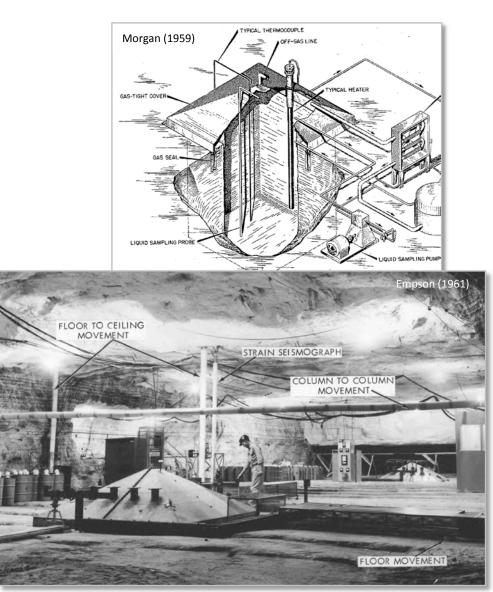
- Lab test in 2' salt blocks (1/10 scale) for 2 months (1959)
- Field test in pits in mine floor (1/5 scale) for 2.5 months (1960)
- Full-scale test in mine floor for 13 months (1961)

Monitored waste/salt behavior

- Room + cavity creep closure
- Solids precipitation/deposition
- Corrosion of materials
- Gas generation

Learned:

 Direct liquid disposal infeasible due to gas generation & cavity stability





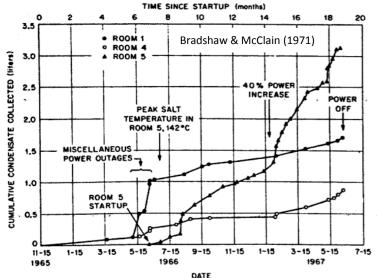
ORNL Project Salt Vault: Lyons, KS

- Lyons AEC solid waste demo
- Hot borehole (July '62)
 - Two 5-kW heaters
 - Salt to >350° C (major decrepitation)
- 3 heater test sites ('65 '67)
 - 7 boreholes per site (10.5 kW)
 - Change out radioactive sources
- Heated Pillar Creep ('66 '67)
 - Driven by 22 heaters (33 kW)

Learned:

- Significant brine from non-salt layers
- Decrepitation can be issue
- Brine inclusion migration to heaters
- Without numerical simulations







OWI/ONWI (RESPEC) Avery Island, LA

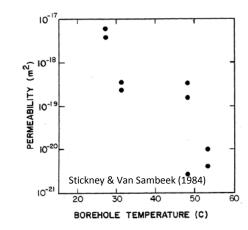
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Site C heater test ('78 – '83)

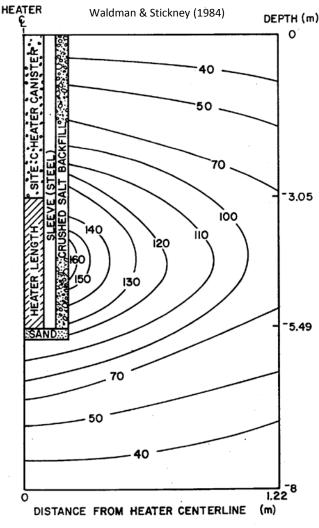
- Central + 8 guard heaters (5.6 kW)
- Heater power constant for 5 years
- Salt k testing using gas flow
- Thermal conductivity (α) salt/backfill

Learned:

Salt k ≈ 10⁴ decrease with heating (healing DRZ) due to creep + thermal expansion









OWI/ONWI (RESPEC) Avery Island, LA

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Brine migration test ('79 – '80)

- Unheated/heated boreholes
- Tracer test (Deuterium)

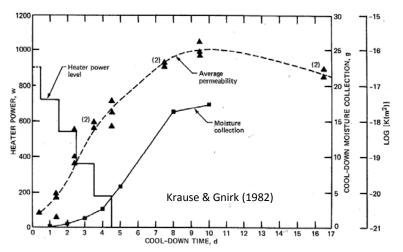
Gas permeability tests

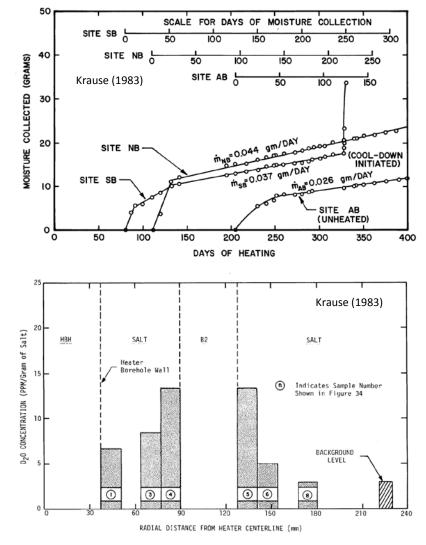
- $k \approx 10^5$ increase during cool down

Learned:

Brine inclusion flow not significant (salt is porous medium)

- k increase @ cooling allows brine flow





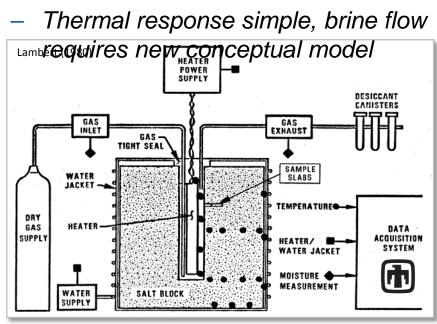


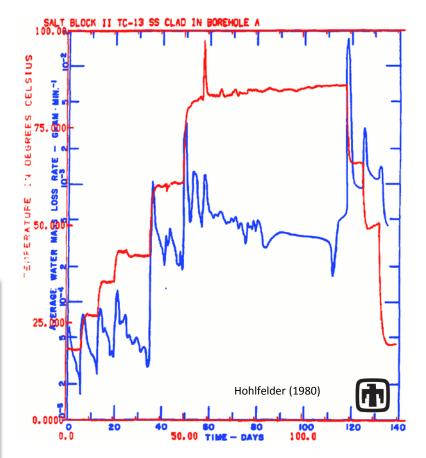
SNL Lab Test: Salt Block II ('78 – '79)

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- 1 m salt cylinder (1700 kg)
- Axially heated/cooled in steps
- High-frequency monitoring
 - Brine inflow to heater borehole
 - Temperature distribution

Learned:





±Temperature changes lead to spikes in brine inflow

Largest spike in brine inflow @ first cool-down step

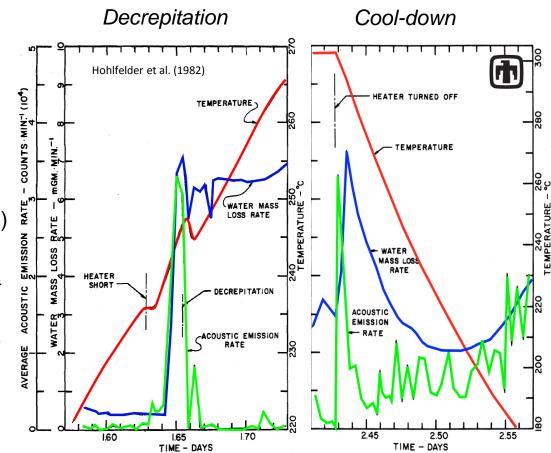


SNL Lab Test: Salt Cracker ('80)

- Two 1.6-kg salt cylinders
- Heated to 200 & 300° C
- Brine Release Events
 - Decrepitation (inclusions)
 - Cool down (increase in k and porosity (n))
 - Increase in heater power (differential thermal expansion)

Learned:

- Acoustic emissions reveal salt microfracturing
- Brine release @ cooling, even after decrepitation





SNL Pre-WIPP Potash Mine Tests ('80 – '81)

BRINE NJECTION

TUBES (4)

GAS

TUBE

SAMPLE

30 cm

ELECTRIC

HEATER-

CANISTER

(10×94cm.)

CANISTER

THERMOCOUPLES

SAMPLES. HOLDER

EMPLACEMENT AND INSTRUMENTATION

MID-POINT

BACKFILL

THERMOCOUPLE

ARRAY CORROSION

ħ

Sandia National

aboratories

1.52m

(SOIN.)

Molecke

ø

Torres (1983)

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Tests conducted in Miss. Chem. Company Potash Mine before 1st WIPP shaft

VERTICAL

SPLACEMEN' GAGE

GAS

SAMPLE

TUBE

A-HEIGHT

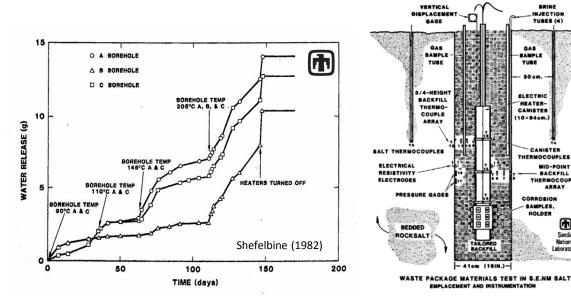
BACKFILL

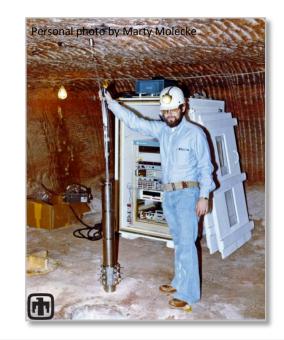
THERMO-

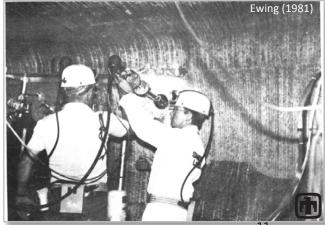
COUPLE

ARRAY

- Waste package material testing
- Heater/brine inflow testing
- Instrumentation "dry run" for WIPP
- Learned:
 - Difficulties of working underground









Waste Isolation Pilot Plant (WIPP)*

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3 Primary SNL DHLW Test Programs

for future Deaf Smith site

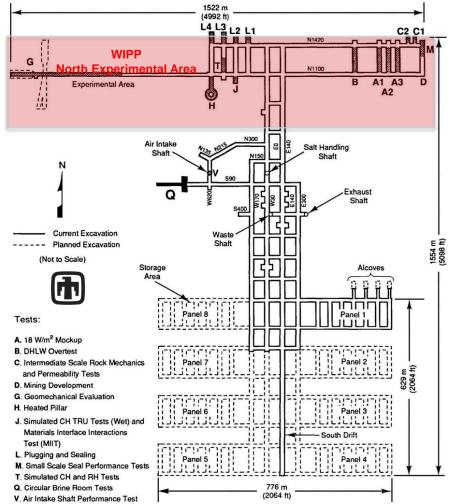
Thermal/Structural Interactions (TSI)

- Rooms A1-A3 (18 W/m² DHLW mockup)
- Room B (DHLW overtest)
- Room H (Heated axisymmetric pillar)
- Room D (Isothermal Room B)

Waste Package Performance (WPP)

- DHLW materials tests in Rooms A1/B
 - Waste Package materials tests
 - Borehole backfill materials tests
- Plugging and Sealing Program (PSP)
 - Brine release in Rooms A1/B
- Many Non-DHLW programs
 - TRU tests Rooms J & T, brine flow in Q, etc.

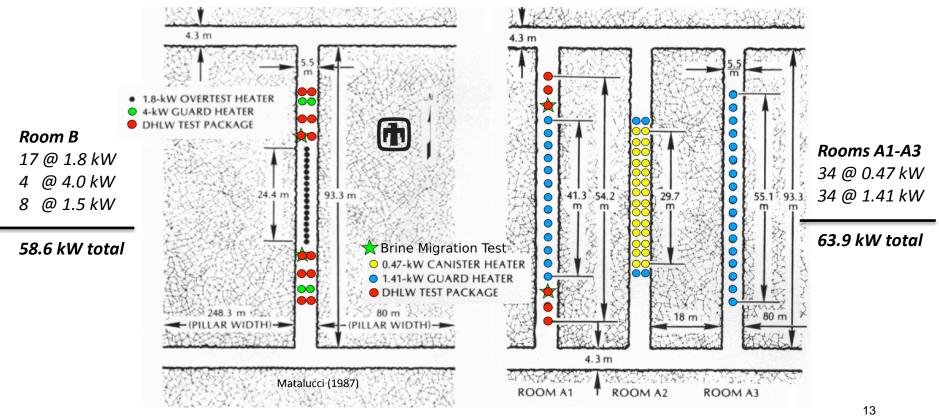
* Tyler et al. (1988) is comprehensive summary of DHLW testing at WIPP



Jensen et al. (1993)



- A Rooms: "design" DHLW thermal load (470 W heaters)
- Room B: "overtest" conditions (1800 W heaters)
- 4 brine migration boreholes
- 18 Waste Package Performance tests (7 retrieved)





SNL WIPP DHLW: Rooms A/B/D

Rooms A/B:

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 Temperature, differential creep, oriented stress (pressure), brine inflow, room closure, heat flux, and heater power.

Room D:

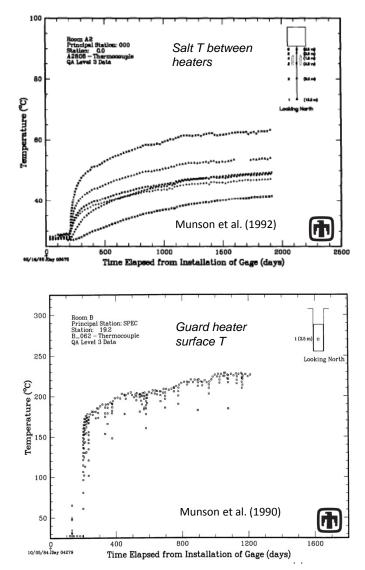
Room B geometry w/ room closure obs.

Learned:

 Roof failure in rooms preceded by rapid closure increase

Room	Mining	Heat on	Heat off
D	Mar-Apr 1984		
В	May-June 1984	Apr 1985	Jan 1989
A2	June-July 1984	Oct 1985	Jun 1990
A1	Sep-Oct 1984	Oct 1985	July 1990
A3	Oct-Nov 1984	Oct 1985	Aug 1990

Kuhlman et al. (2012)





SNL WIPP DHLW: Room A2 ('85-'90)

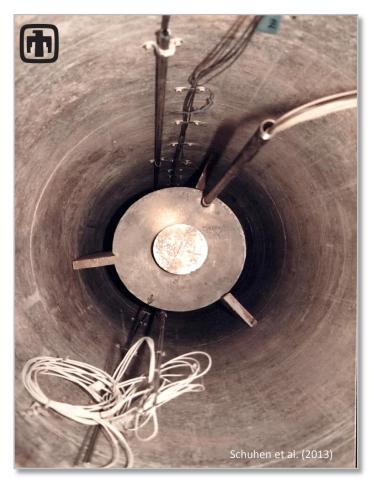
Nuclear Energy

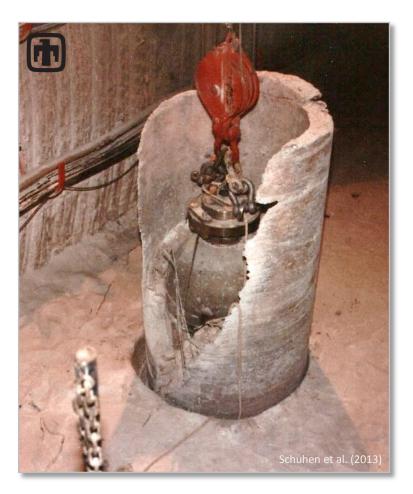




SNL WIPP DHLW: Room B ('85-'89)

Nuclear Energy





Typical WPP DHLW canister in Room B at installation and removal Creep closure and salt crust deposition required overcoring to remove



SNL WIPP DHLW: A1/B Brine Release

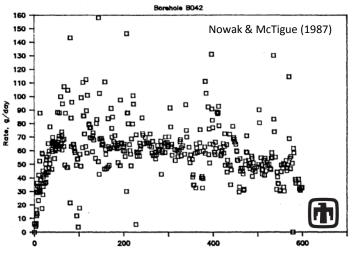
Nuclear Energy

Brine release:

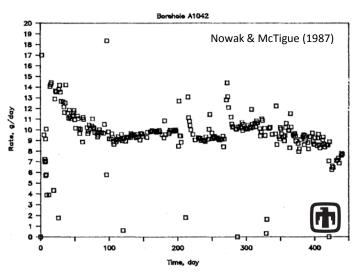
- Quantified before and during heating.
- Room B produced ≈8× more brine from same geology @ ≈3× T
- Significant brine inflow at Clay F

Learned:

- Vapor transport of brine in intact salt is insignificant
- Observed brine inflow consistent with salt rind observed @ heater retrieval.
- Thermo-poro-elasticitic model (McTigue, 1990) consistent with observed heated and isothermal flow (didn't consider brine inclusions)



Room B: 130° C, 35 L brine/borehole Room A1: 50° C, 4.5 L brine/borehole





ANDRA Amélie Tests* ('87 – '94)

Heated borehole consolidation of crushed salt ('87–'88)

5-boreholes: different grain-size distributions
1.6 & 2.2-kW heaters

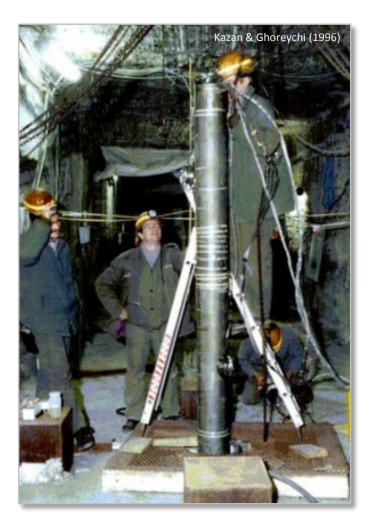
Borehole heater test (CPPS) ('91–'93)

- 4-kW heater in 7-m borehole
- Reached 200° C max T at heater
- 7 months of heating

Gas/brine permeability tests ('94)

Learned:

- No backfill complicated heat transfer, while crushed salt simplified it
- Viscoplasticity model needed to explain brine flow under some conditions





Asse II Mine

Borehole heater tests 1–6 ('68 – '85)

- Early tests to
 - Determine in situ thermal properties of halite / crushed salt
 - Demonstrate heater, thermocouple, and brine collection systems
 - Demonstrate geophysical methods to interrogate heated salt (Kessels et al., 1986)
- Heated deep borehole closure ('79 '82)
 - Closure data (calipers) inside borehole during heating (Doeven et al., 1983)
- Heated Brine Migration test ('83 '85)
- High Activity Waste (HAW) heater test ('88 '94)
- Crushed Salt Reconsolidation
 - Heated drift backfilled with crushed salt: TSDE ('90 '04)
 - Heated vertical boreholes: DEBORA-1/2 ('97 '98)



Asse Brine Migration

Heated 4 borehole sets ('83 – '85):

- 2 with ⁶⁰Co sources
- 2 sealed (vs. 1 atm)

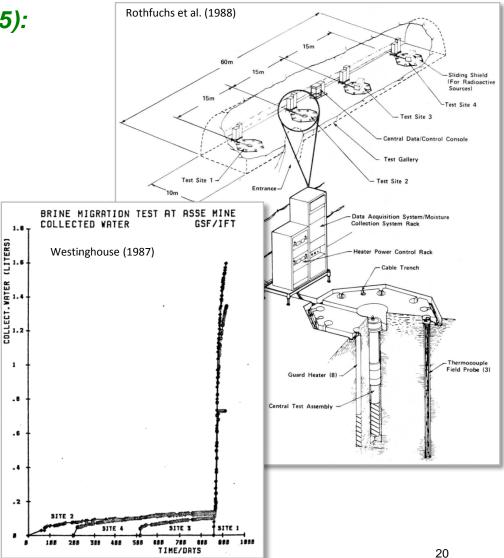
Measured

- Closure, Temperature
- Brine inflow
- Borehole gas content
- Acoustic emissions

90% of brine collected during cooling

Learned:

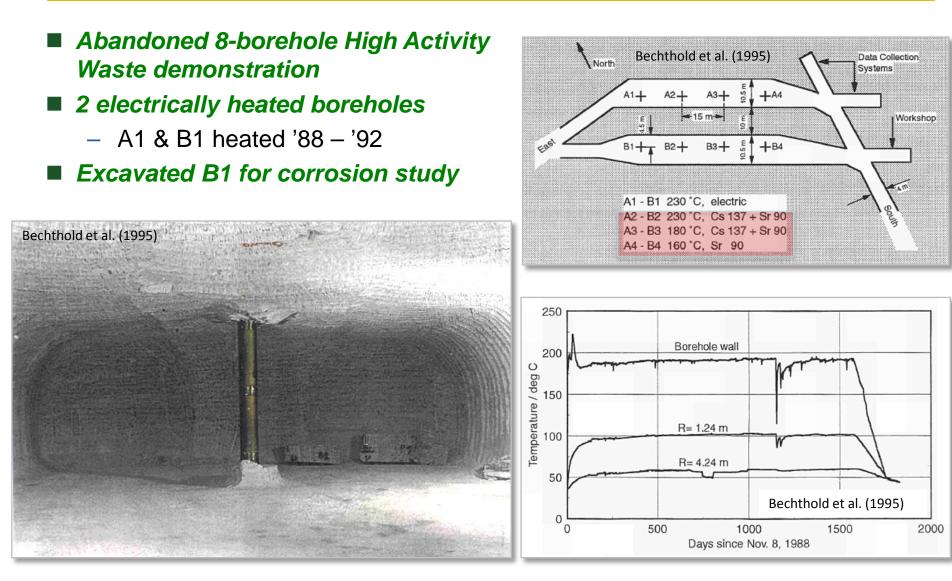
- Mechanical similar to bedded, brine flow << bedded
- Radiation had minimal effect





Asse HAW Heater Tests

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Asse Borehole Salt Reconsolidation

Nuclear Energy

Measured

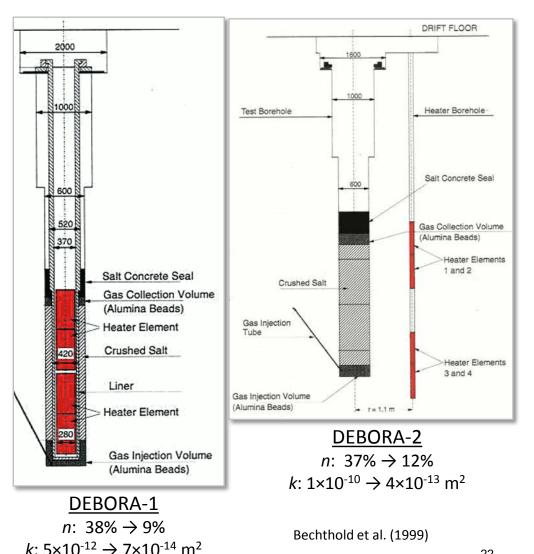
- Corrosion,
- Temperature, pressure,
- Borehole convergence,
- Crushed salt k & n

DEBORA-1 ('97 – '98)

- 9-kW heater <u>in</u> 15-m borehole
- DEBORA-2 ('97 '98)
 - 15-kW heaters <u>around</u> 15-m borehole

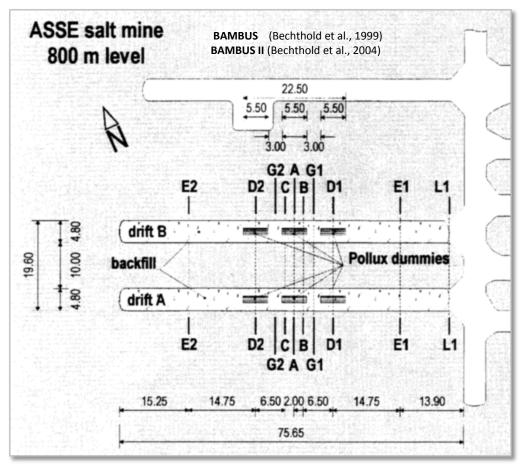
Learned:

 Crushed salt reconsolidated significantly in months in boreholes





- Thermal Simulation of Drift Emplacement (TSDE)
- Six Pollux casks
 - 1.5 m × 5 m
 - 6.4-kW heaters
- Backfilled to roof
- Large thermal-mechanical time series collected
- Post-test excavation data
- Learned:
 - Crushed salt reconsolidation less than in boreholes
 - Extensive in situ validation dataset

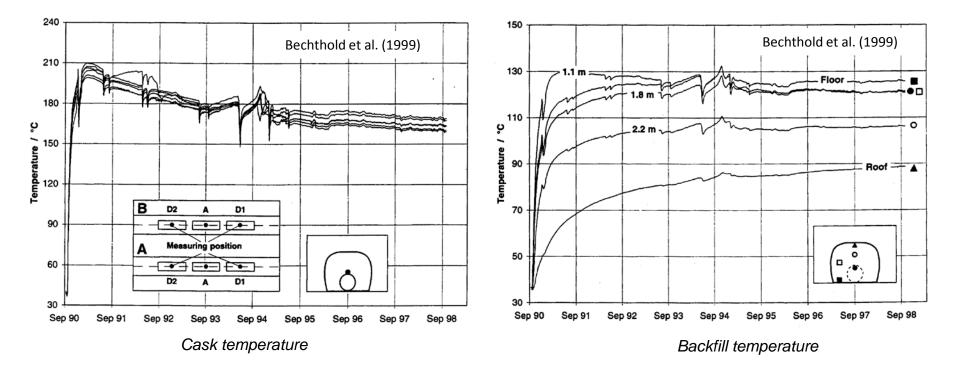




Heater T decreased (at constant power)

- Backfill α increasing with decreasing porosity
- Non-linear thermal conductivity: $\alpha(T) \approx \alpha_0(T T_0)^{-1.1}$

Steady-state T reached near heaters (but not at roof)





Salt Vault presented 1st safety case for HLW in salt (1971)

- Culmination of 10+ years of laboratory and field testing
- Bradshaw & McClain (1971) summarized technical basis
- AEC proposed Lyons, KS as pilot-scale site for heat-generating waste
- NRC Summary
 - Geoscience database for nuclear waste repositories (salt, granite, clay) (Isherwood, 1981)
- Deaf Smith Site Characterization Plan (DOE, 1988; 10 vol.)
 - Regional site description
 - Salt properties determined from core & other sites
 - Site Investigation Plan (very detailed)
- Gorleben safety case: ISIBEL project (2006—2010)
 - Site characterization
 - Disposal system design
 - Weber et al. (2011) summarized safety case.
- Recent UFD/SNL historical testing and technical basis summaries
 - Kuhlman et al. (2012), Kuhlman & Malama (2013), Kuhlman & Sevougian (2013)



Technical basis fo	r heat-generating	waste in
salt is <u>not new</u>		

- Thermal-mechanical behavior is well known
- Modern numerical models
 - Allow non-linear and coupled processes
 - Must be benchmarked against data
 - Not technical basis, but important tools

• Long-term viability of salt repository:

- Salt deposit provides long-term containment
- Shaft seals ensure containment uncompromised
 - Seal emplacement
 - Reconsolidation of backfill
- Other repository features of secondary safety case importance
 - Waste forms/waste packages
 - Brine migration into and through excavation

Test Locale	<u>B</u> edded vs. <u>D</u> omal	<u>C</u> rushed vs. <u>I</u> ntact	<u>B</u> orehole vs. In- <u>D</u> rift
Salt Vault	В	Ι	В
Avery Island	D	Ι	В
WIPP DHLW	В	I	В
Amélie	В	C + I	В
Asse	D	C + I	B + D
?	В	С	D



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