



# SNF Cladding Hydride Reorientation Research

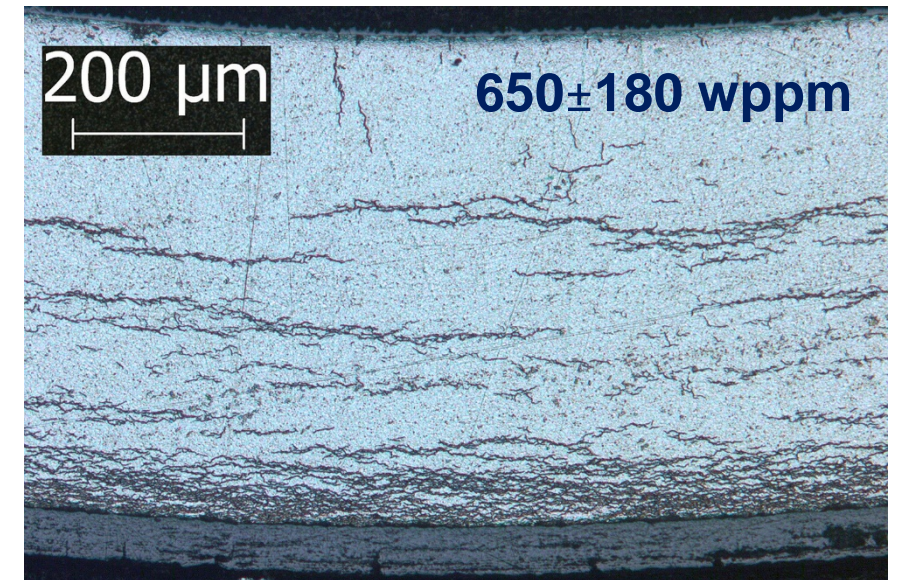
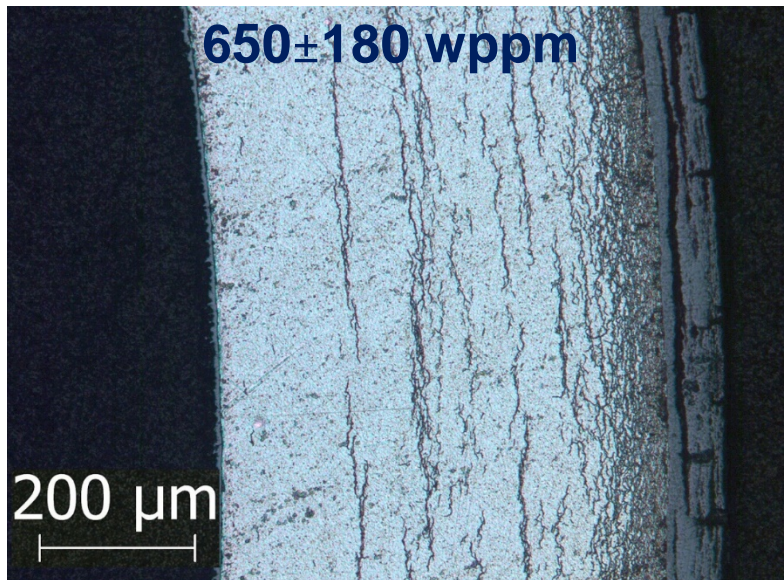
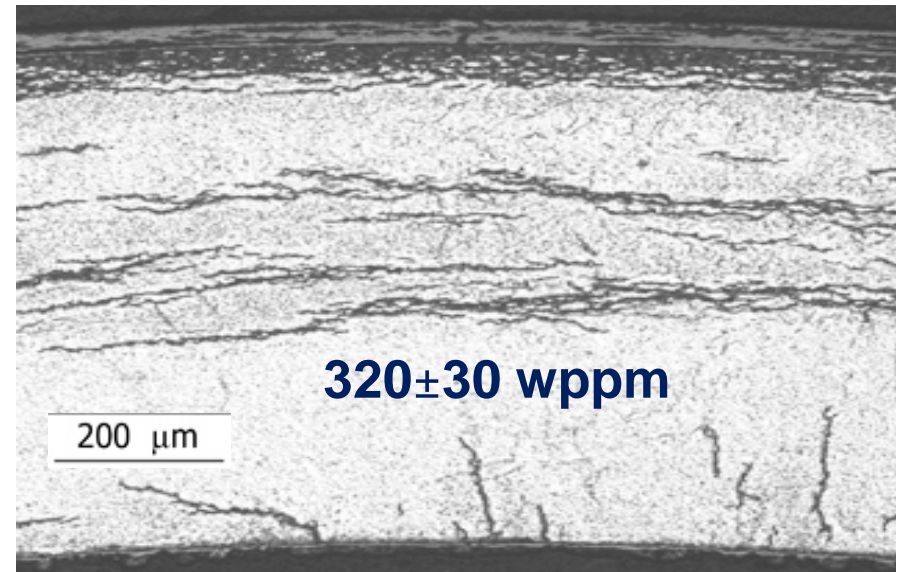
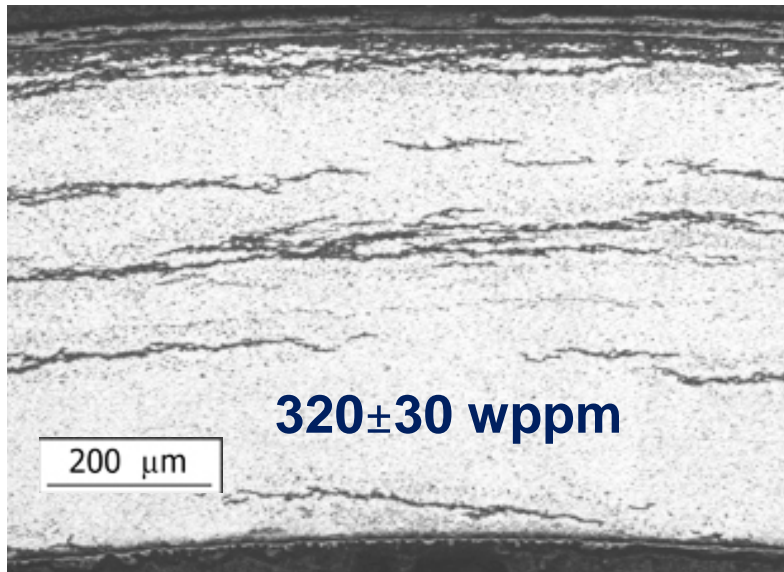
M.C. Billone

NWTRB Meeting  
Albuquerque, NM  
Oct. 24, 2018

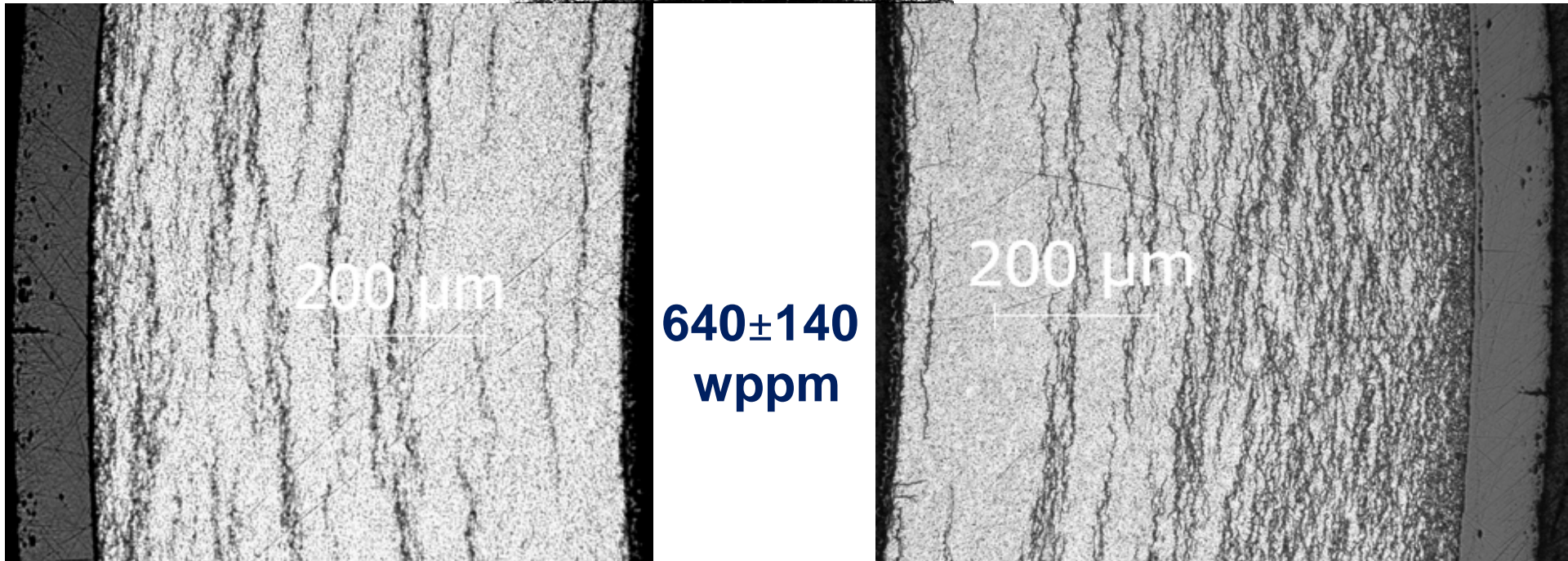
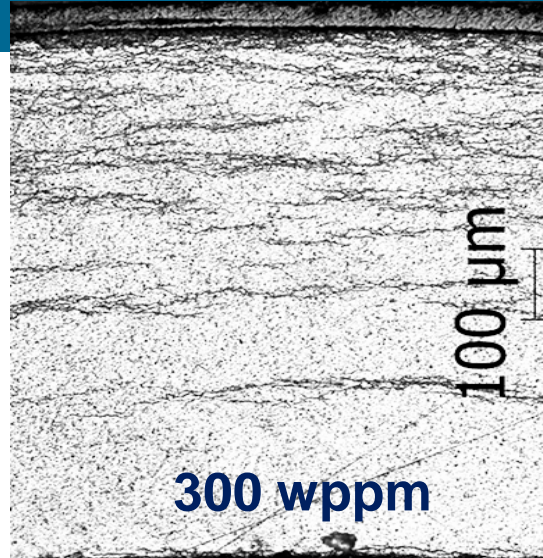
# Background on Cladding and Zirconium Hydrides

- **PWR Cladding Alloys and Thermal-Mechanical Treatment**
  - Alloys:
    - Zircaloy-4 (Zr-1.5wt.%Sn), Low-Sn Zircaloy-4 (Zr-1.3 wt.%Sn)
    - ZIRLO<sup>®</sup> (Zr-1wt.%Sn-1wt.%Nb), M5<sup>®</sup> (Zr-1wt.%Nb)
  - Thermal-mechanical treatment (final heat treatment)
    - Recrystallized-annealed microstructure ( $600 \pm 20^{\circ}\text{C}$  for 1 to 2 hours)
    - Cold-worked stress-relief-annealed microstructure (e.g.,  $500^{\circ}\text{C}$  for 4 hours)
- **Hydrogen Content and Hydride Orientation in As-Irradiated Cladding (e.g., prior to drying and storage)**
  - ZIRLO<sup>®</sup> with 320 & 650 weight parts per million (wppm) hydrogen (H)
  - Zircaloy-4 with 300 & 640 wppm H
  - M5<sup>®</sup> with 76 wppm H (peak values for M5<sup>®</sup> are  $100 \pm 30$  wppm H)

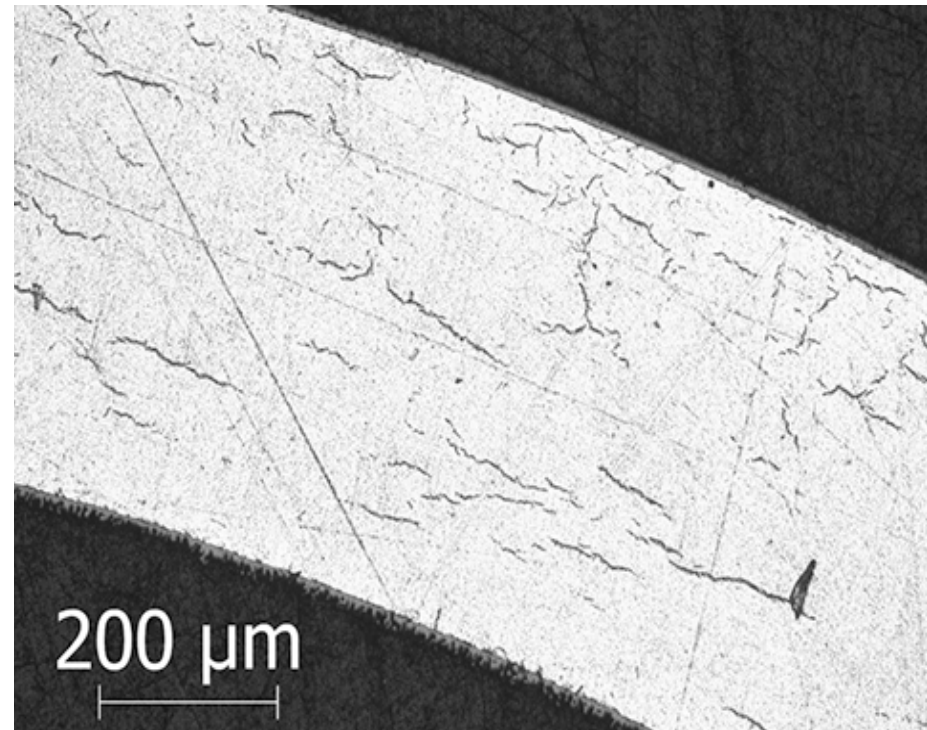
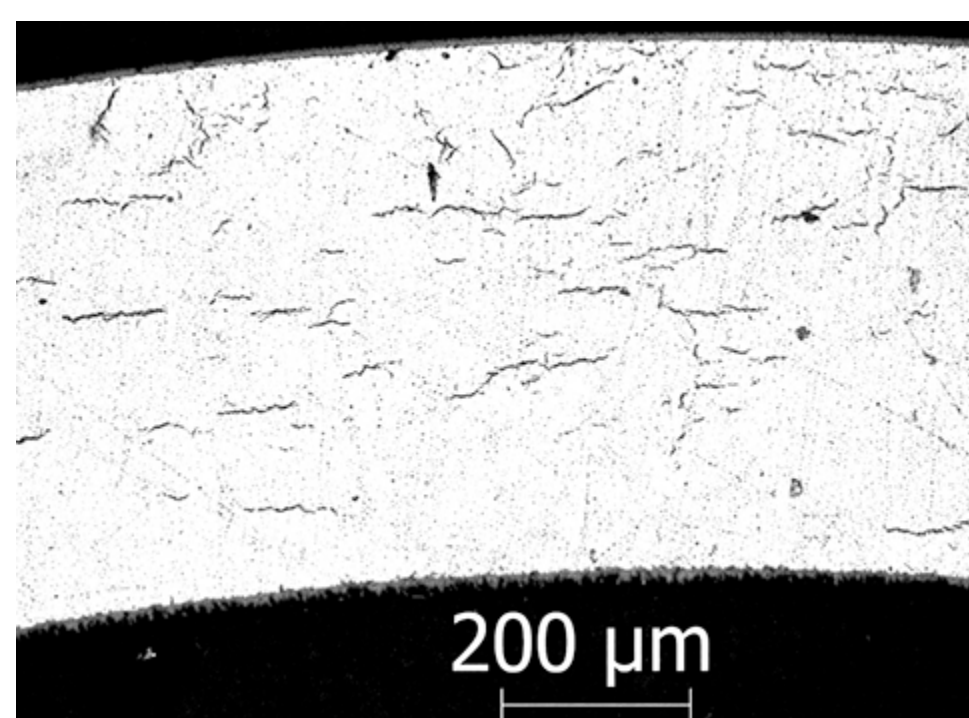
# Hydride Orientation in Irradiated ZIRLO<sup>®</sup> Cladding



# Hydride Orientation in Irradiated Zircaloy-4 Cladding



# Hydride Orientation in Irradiated M5<sup>®</sup> Cladding



**76±5 wppm**

# Objectives of Phase 1 Sister Rod Testing

- **Characterization and Material Properties**
  - Generate data that can be compared to 10-year stored PWR fuel rods: baseline data for as-irradiated fuel rods
  - Determine end-of-life rod internal pressures (limited public database), mechanical properties of M5<sup>®</sup> (published data are inadequate) and mechanical properties of ZIRLO<sup>®</sup> (limited public database)
- **Is Radial-Hydride-Induced Embrittlement an Issue?**
  - Use measured rod-internal pressures ( $P_i$ ) at 25°C
  - Heat treat @ 400°C [ $\leq 200$  wppm H in solution;  $2.26 \times P_i(25^\circ\text{C})$ ]
  - Peak cladding hoop stresses @ 400°C: reasonable upper bound for standard PWR stored rods (average gas temperature is  $< 400^\circ\text{C}$ )
  - Cool at  $\leq 5^\circ\text{C}/\text{h}$  with decreasing pressure/hoop-stress
  - Perform ring compression tests to determine ductility and ductility transition temperature (DTT)

# Potential for Hydride Reorientation

- **Radial-Hydride-Induced Degradation**

- DEMO Cask fuel rods

- Peak cladding temperature (PCT)  $<250^{\circ}\text{C}$ ;  $<44$  wppm H in solution
- Internal gas pressure  $<9$  MPa; peak hoop stress  $<68$  MPa
- May observe short radial hydrides, but these would not decrease ductility

- Sister Rods

- PCT =  $400^{\circ}\text{C}$ ;  $\leq 200$  wppm H; pressure  $<11.3$  MPa; hoop stress  $<87$  MPa
- Expect ZIRLO<sup>®</sup> & Zry-4 radial hydrides  $<20\%$  of cladding wall; DTT  $<50^{\circ}\text{C}$
- Expect longer radial hydrides in M5<sup>®</sup>; DTT  $<75^{\circ}\text{C}$

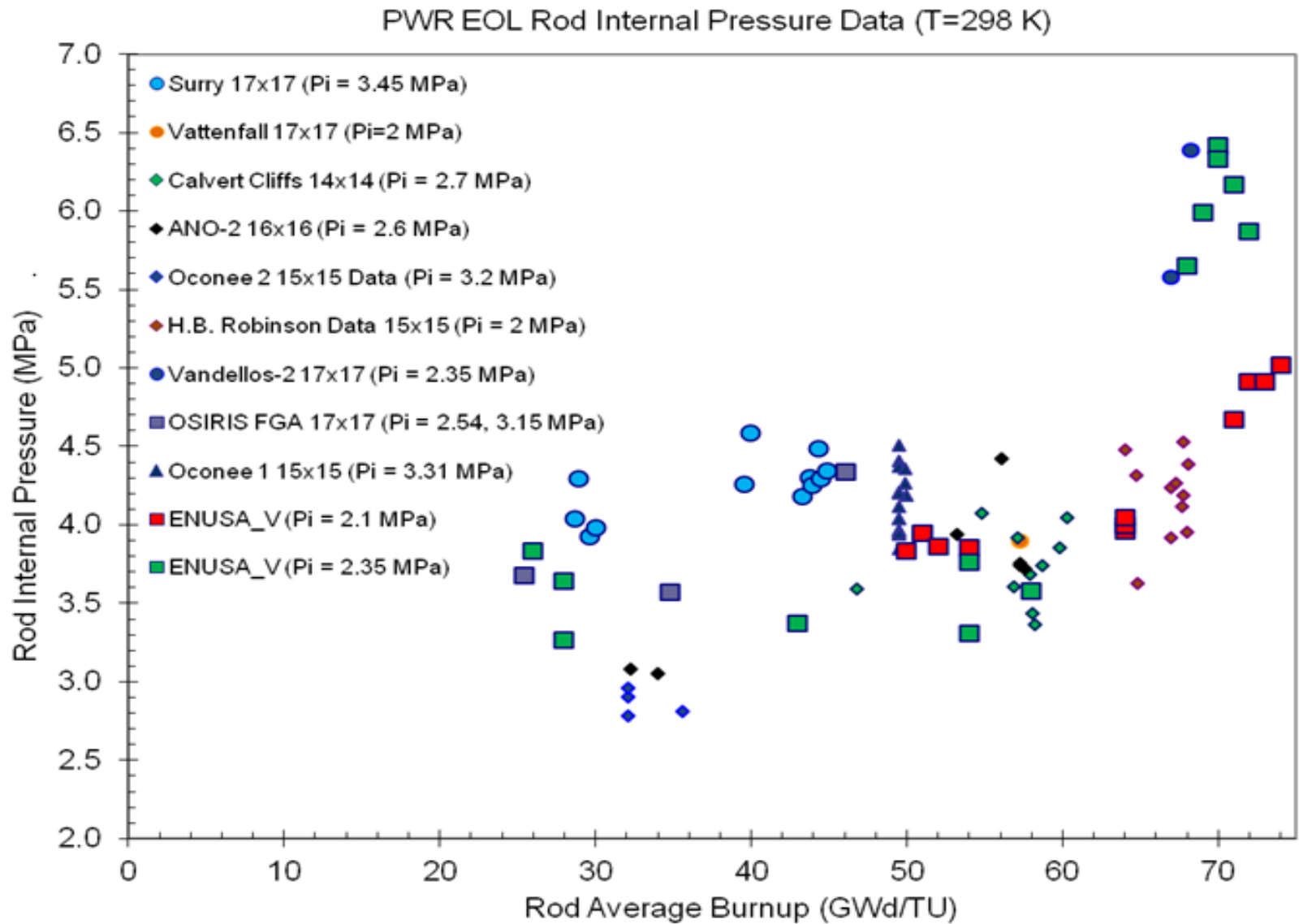
- **Anticipated Range of Peak Cladding Hoop Stresses**

- FRAPCON predictions for  $400^{\circ}\text{C}$  PCT and temperature profile

- $<54$  MPa for standard PWR rods;  $<89$  MPa for IFBA rods with internal B-10

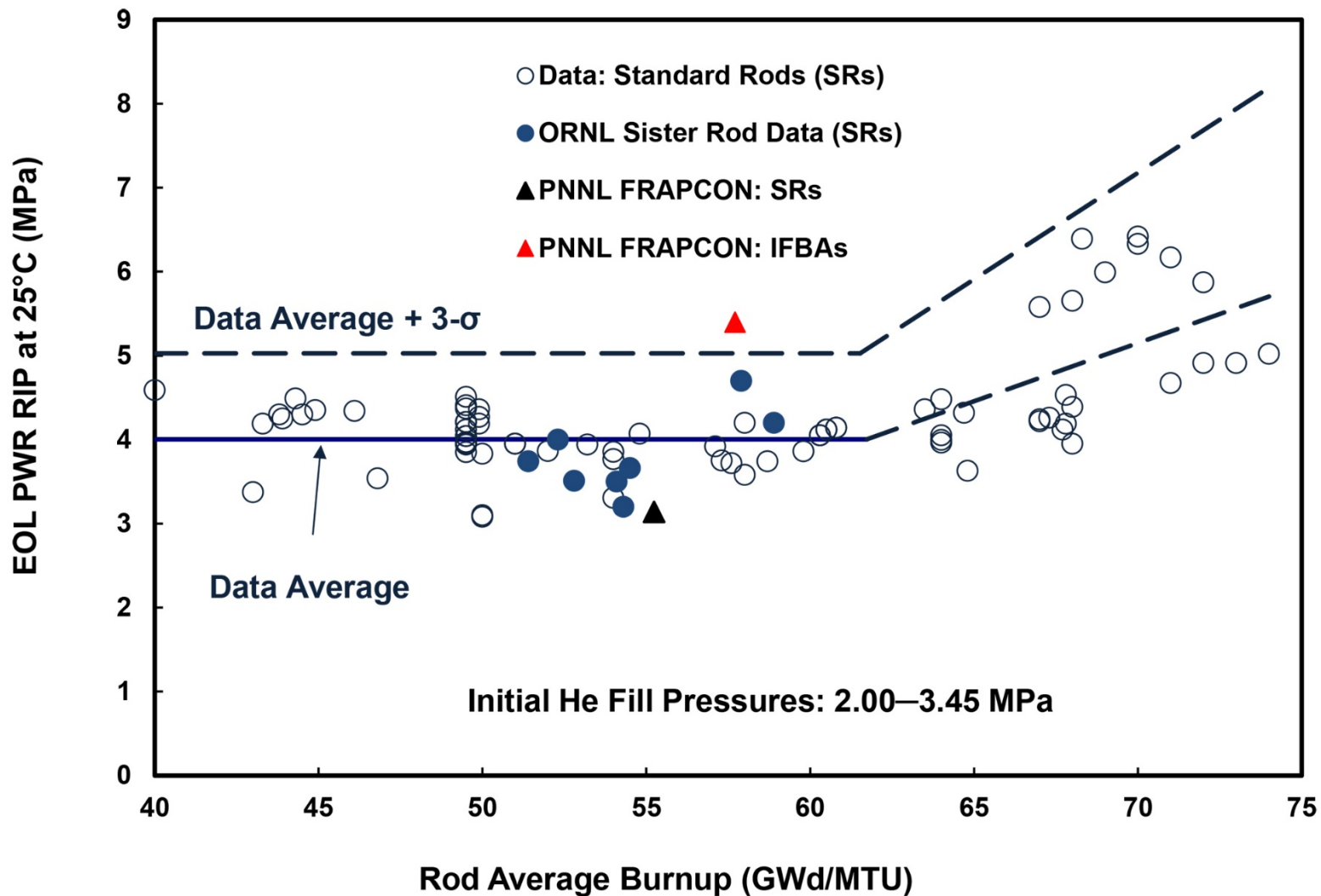
- Predictions based on end-of-life (EOL) rod internal pressure (RIP) data

# EPRI Data for EOL RIP





# EOL RIP EPRI & Sister Rod Data



# Relationship between Hoop Stress and Pressure

- **Definition of Parameters**

- $P_i$  = internal rod pressure,  $P_o$  = external rod pressure (0.0–0.7 MPa)
- $R_{mi}$  = cladding inner radius,  $h_{mi}$  = cladding wall thickness

- **Average Hoop Stress ( $\sigma_\theta$ ) Across Cladding Wall**

- $\sigma_\theta = (R_{mi}/h_{mi}) (P_i - P_o) - P_o$

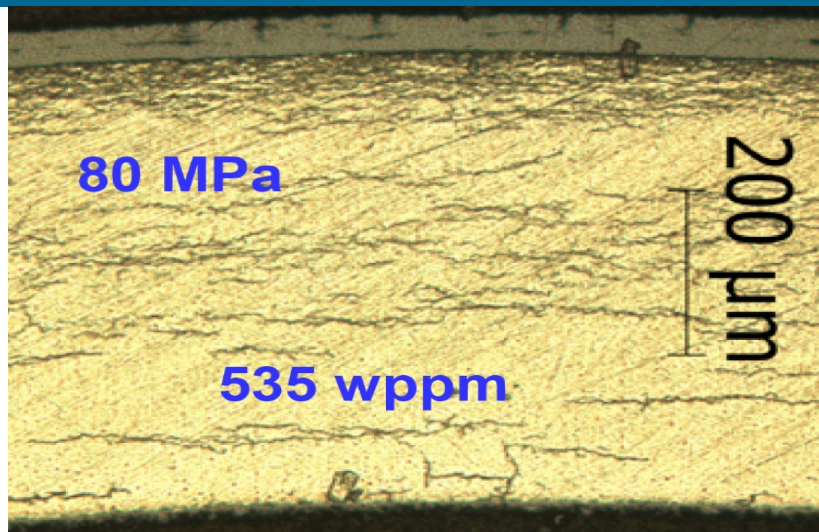
- **Changes in Parameters with Burnup (BU)**

- $P_i$  increases with BU due to free volume decrease & He release (IFBA)
- $R_{mi}$  decreases with BU up to  $\approx 40$  GWd/MTU then increases
- $h_{mi}$  decreases with BU due to coolant-side oxidation
- $R_{mi}/h_{mi} = 7.3 \rightarrow 7.7$  (60- $\mu\text{m}$  oxide layer)  $\rightarrow 8.3$  (100- $\mu\text{m}$  oxide layer)
  - Initial values for  $17 \times 17$  array: 4.18-mm inner radius, 0.57-mm wall

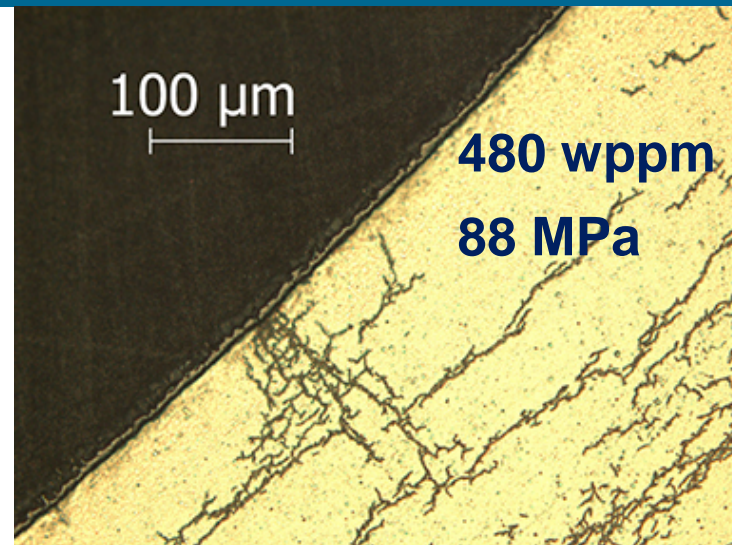
# Review of Hydride-Induced Ductility Degradation

- **As-Irradiated Cladding**
  - Hydrides are primarily oriented in circumferential direction
  - Distribution of circumferential hydrides across cladding wall has a significant effect on ductility & radial-hydride precipitation
  - Short isolated radial hydrides have been observed in cladding (ZIRLO<sup>®</sup> and M5<sup>®</sup>) from PWR fuel rods irradiated to high burnup
- **Conditions for *Significant* Radial Hydride Precipitation**
  - High enough (>60 wppm) hydrogen content and peak cladding temperature (PCT >285°C)
  - High enough (>10 MPa) internal pressure at PCT
  - High enough (>80 MPa: ZIRLO<sup>®</sup> and Zry-4) hoop stress @ PCT
  - Cladding microstructure (RXA more susceptible than CW-SRA)
  - Distribution of circumferential hydrides @ PCT
- **ANL Results for 400°C & 350°C PCT**

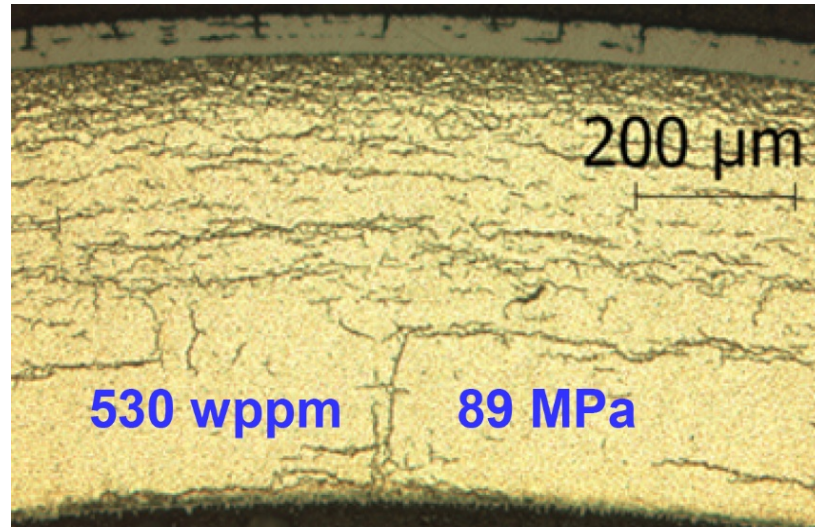
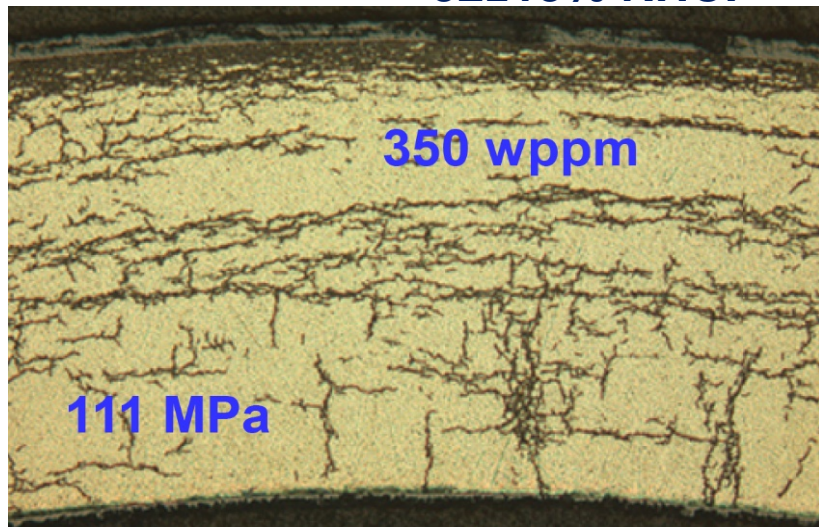
# Radial Hydrides in ZIRLO<sup>®</sup> for 400°C PCT



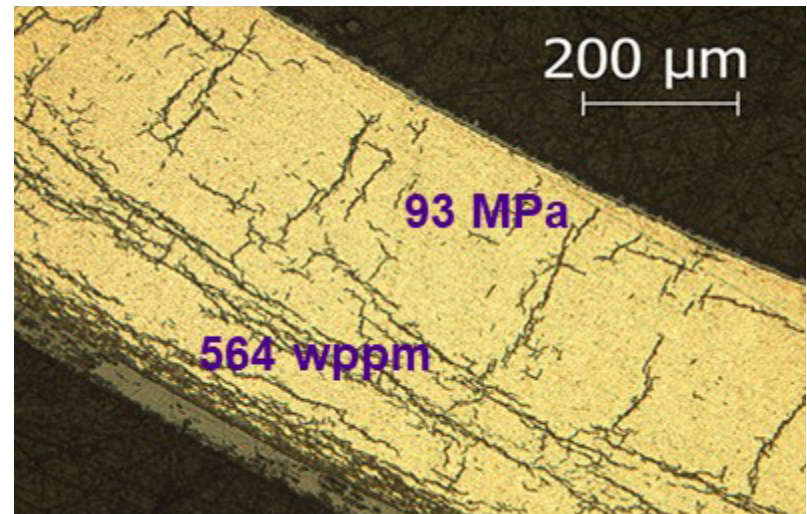
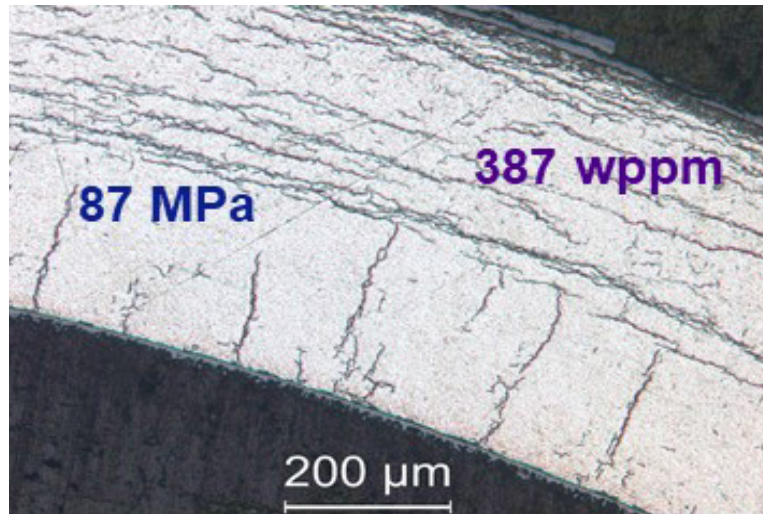
<20°C DTT      9±4% RHCF  
120°C DTT      32±13% RHCF



20°C DTT      20±10% RHCF

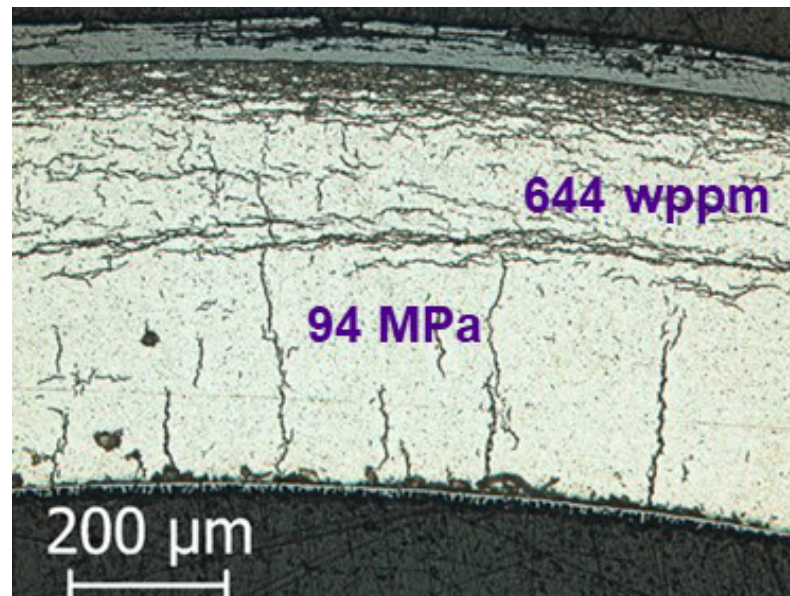


# Radial Hydrides in ZIRLO<sup>®</sup> for 350°C PCT



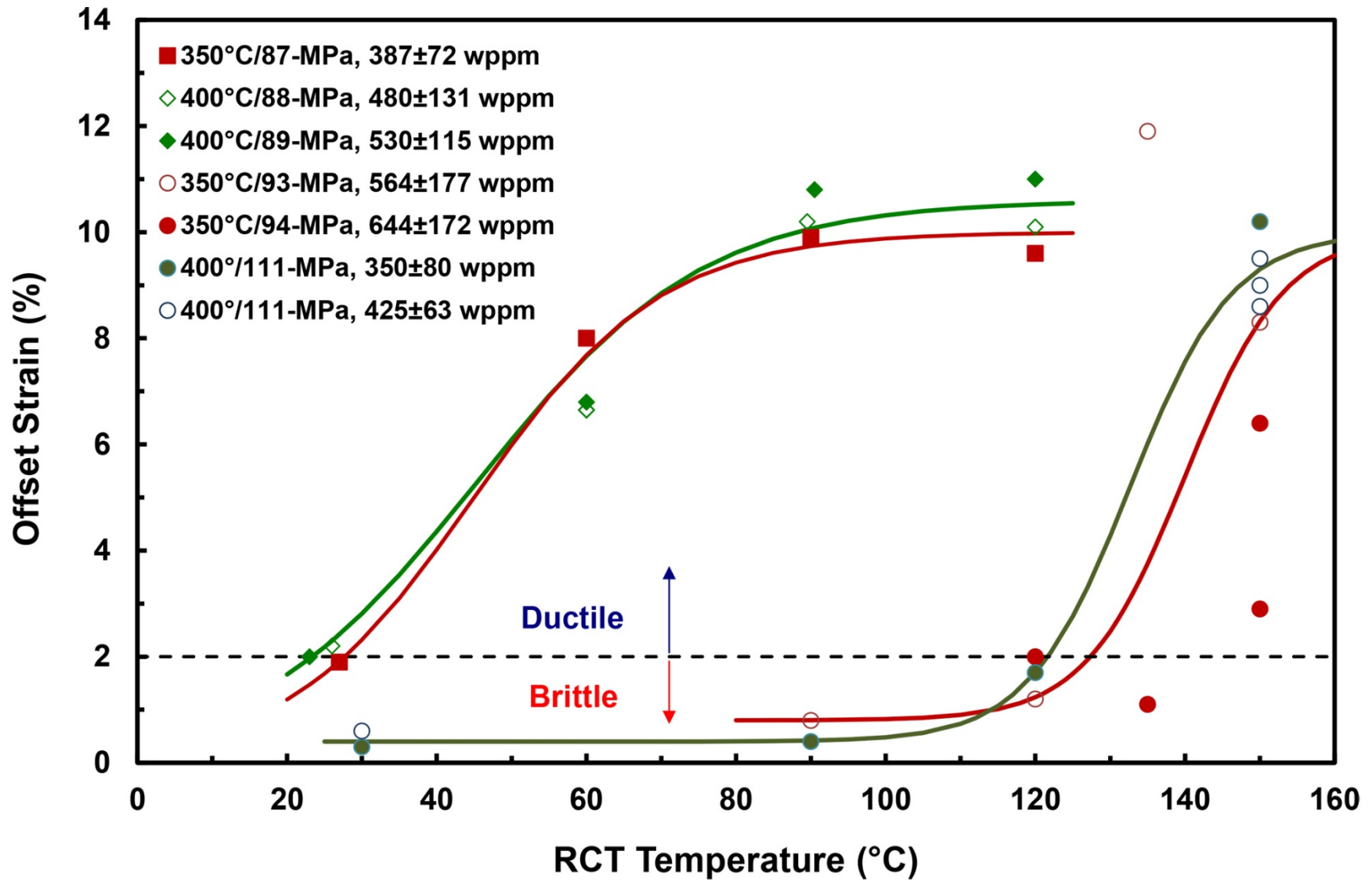
**28°C DTT**  
**20±10% RHCF**

**≈125°C DTT**  
**30±11% RHCF**

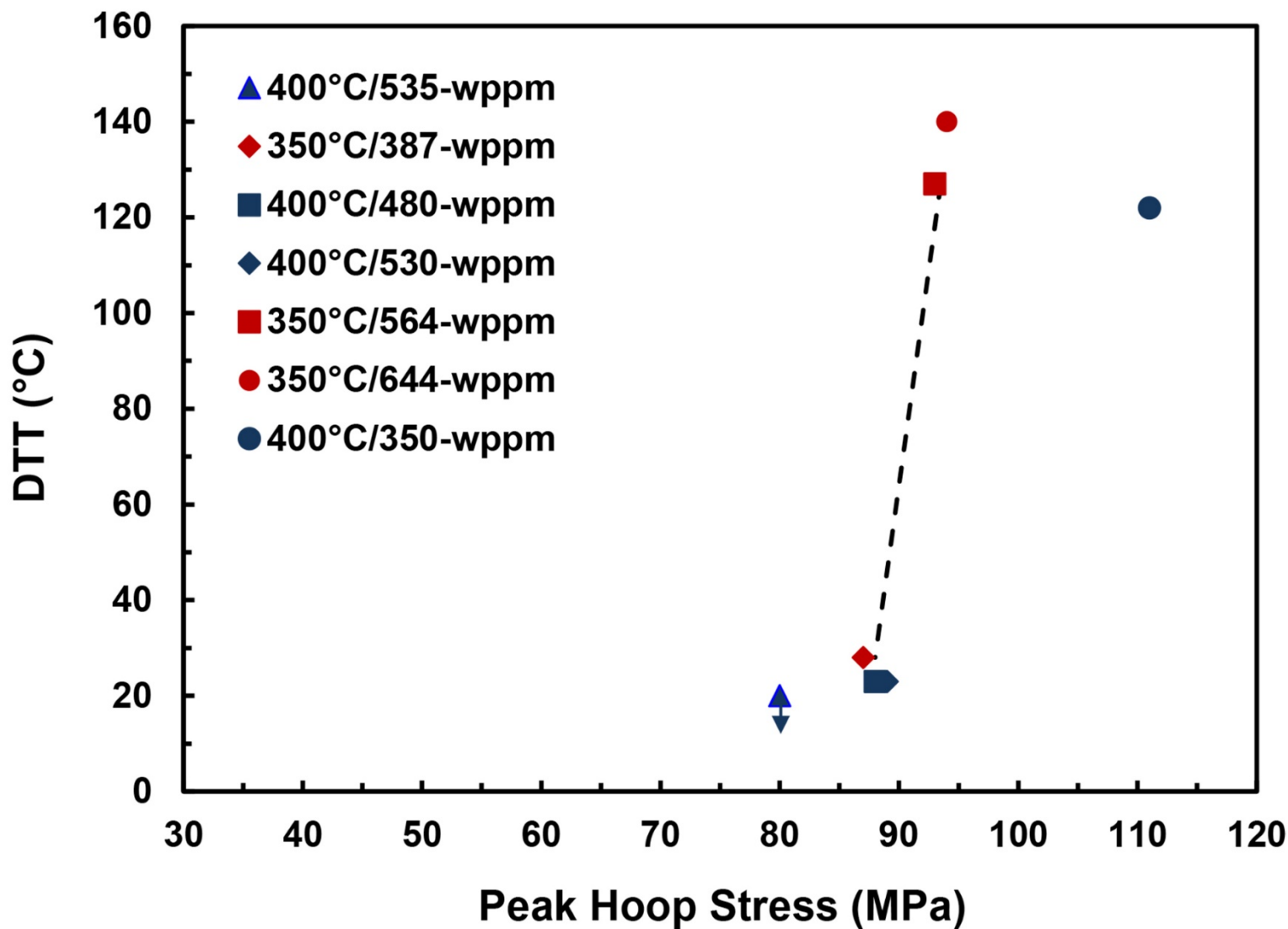


**≈140°C DTT**  
**37±11% RHCF**

# Ductility (Offset Strain) for ZIRLO<sup>®</sup> vs. Test Temperature – DTT Determination



# Sensitivity of Irradiated ZIRLO<sup>®</sup> to Peak Cladding Hoop Stress



# Future Hydride-Reorientation Testing

- **ANL Cladding at 350°C PCT**
  - Critical stress for ZIRLO<sup>®</sup> appears to be  $90 \pm 3$  MPa
  - ZIRLO<sup>®</sup>: test 350-wppm-H/93-MPa, 650-wppm-H/87-MPa
  - ANL cladding is from lead-test-assembly rods at >62 GWd/MTU
- **Sister Rod Cladding at 400°C PCT**
  - More prototypic linear power histories per fuel cycle (18 months)
  - Wider range of hydride distributions through cladding wall
  - More cladding samples available
    - Allows for repeat tests and intermediate-temperature tests
    - Allows for load-interrupt tests to better determine cladding ductility
    - Offers the possibility of M5<sup>®</sup> with >100 wppm hydrogen
- **M5<sup>®</sup>, Ring Compression Tests, Offset Strains, etc.**
  - Possible discussion items

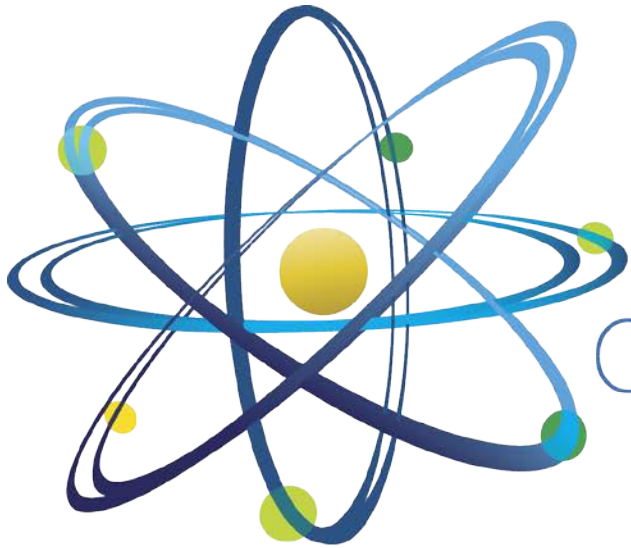


# Acknowledgement

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# Questions?



Clean. **Reliable. Nuclear.**

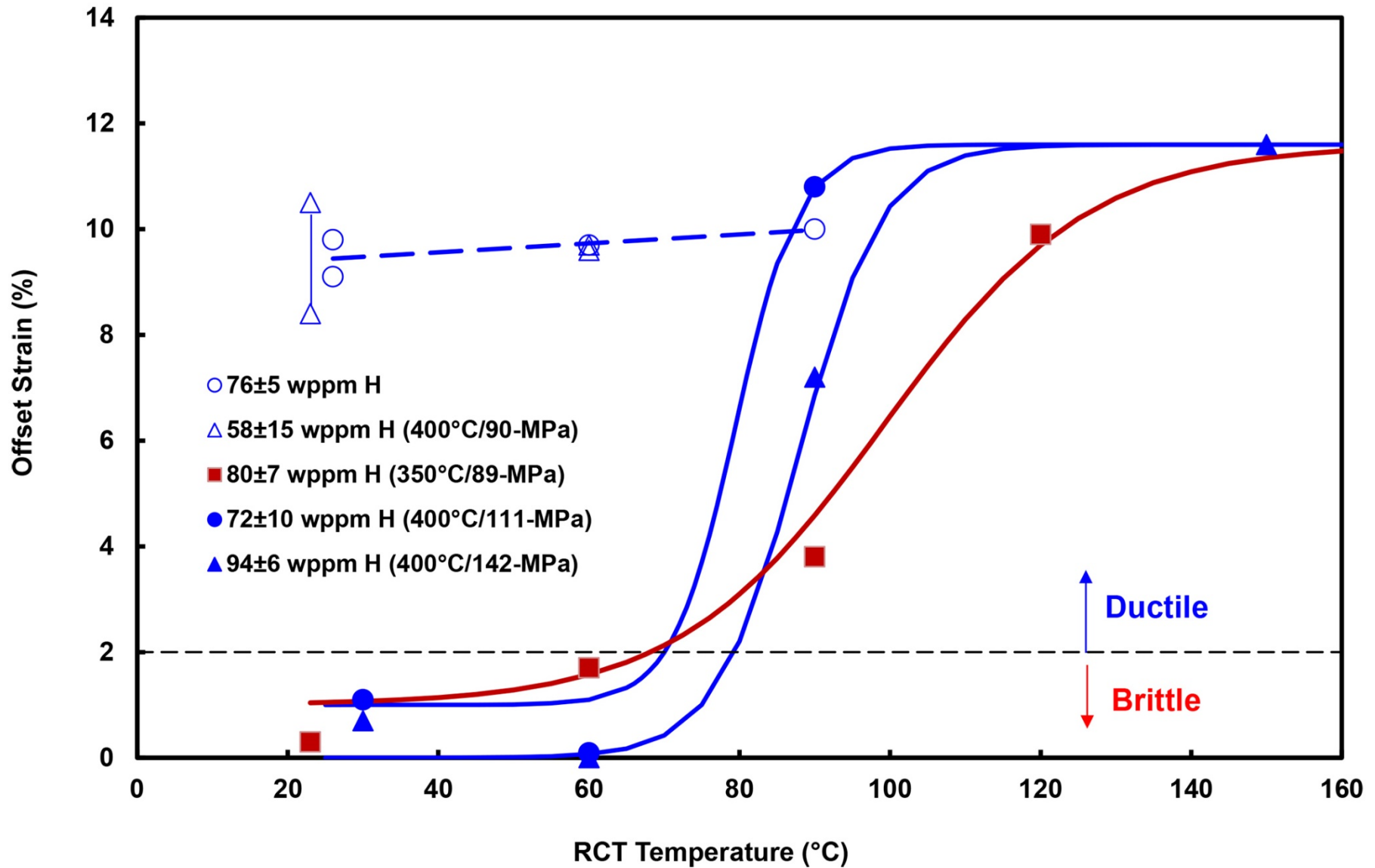
- **Results for Irradiated M5<sup>®</sup> Cladding**

- Conditions and results for high ductility with <20°C DTT
  - As-irradiated with  $76 \pm 5$  wppm H
  - $58 \pm 15$  wppm, 400°C/90-MPa,  $37 \pm 17\%$  RHCF
- Conditions for ductility transition at  $75 \pm 5^\circ\text{C}$ 
  - $80 \pm 7$  wppm, 350°C/89-MPa,  $44 \pm 18\%$  RHCF
  - $72 \pm 10$  wppm, 400°C/111-MPa,  $54 \pm 20\%$  RHCF
  - $94 \pm 6$  wppm, 400°C/142-MPa,  $61 \pm 18\%$  RHCF

- **Observations**

- For <100-wppm H, axial continuity of radial hydrides is limited
- No ductility degradation observed for <60 wppm H
- Need to test M5<sup>®</sup> cladding with 100–130 wppm H

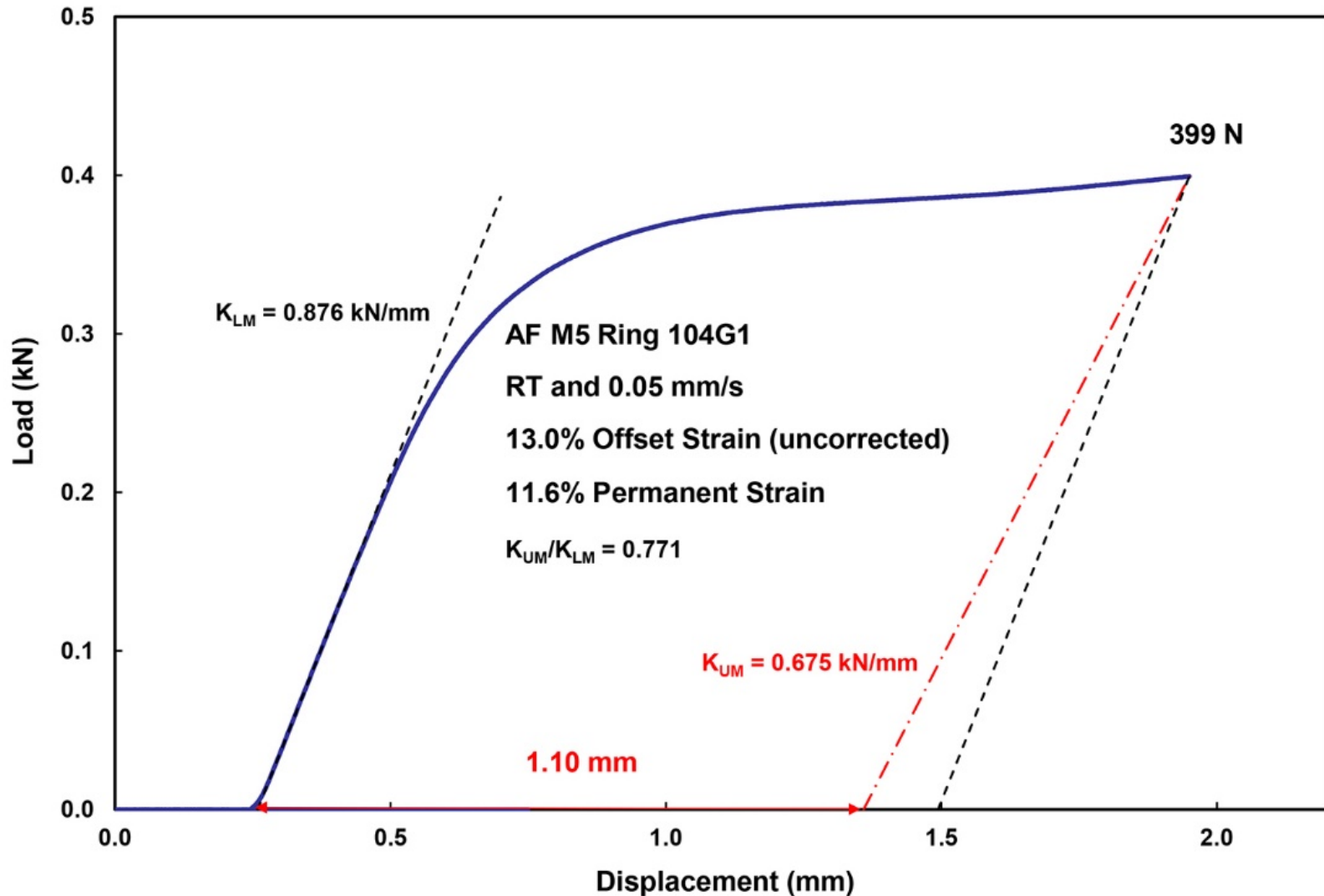
# Ductility Data and DTT for Irradiated M5<sup>®</sup>



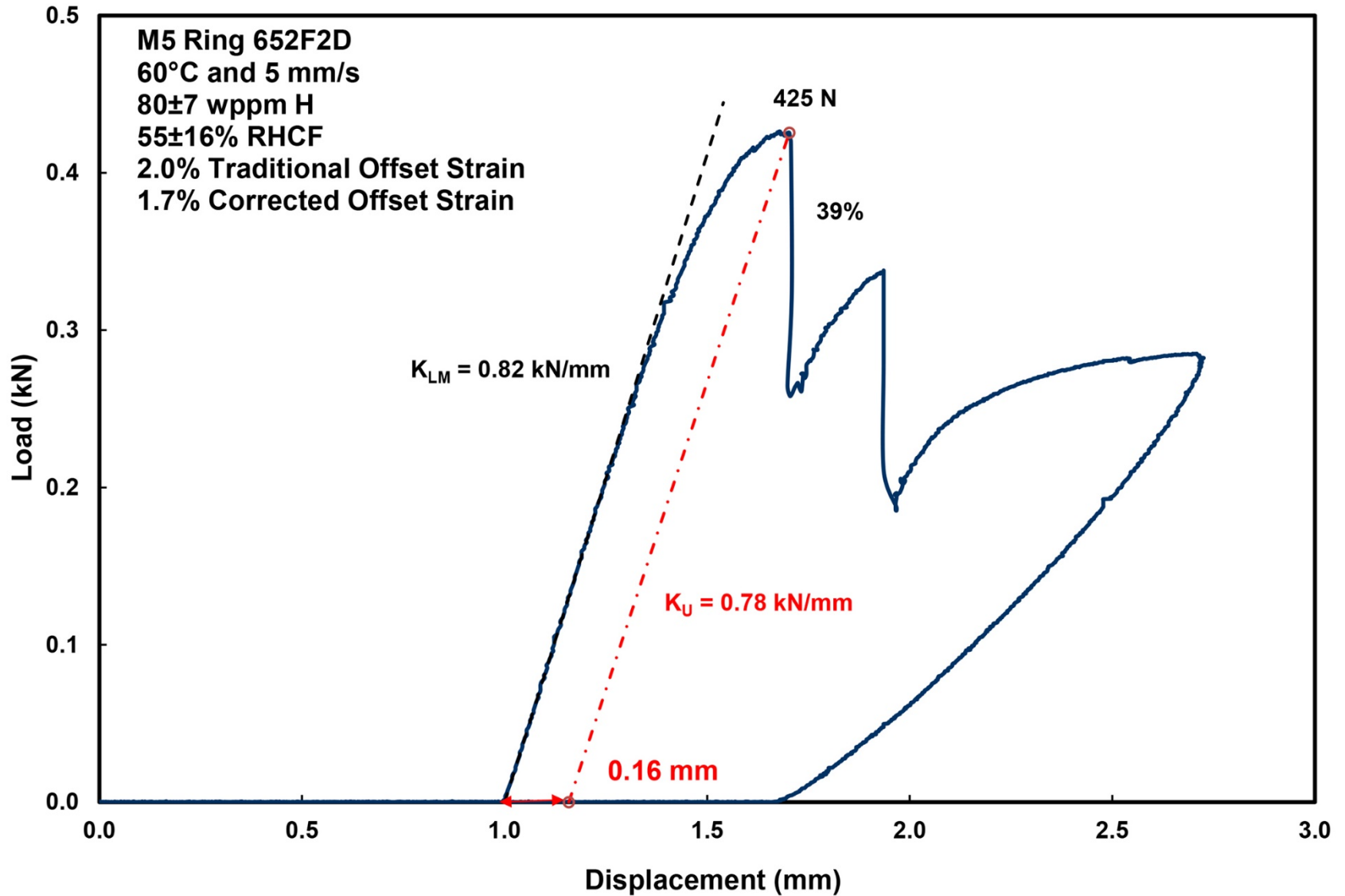
# Ring Compression Tests

- **Ring Compression Tests (RCTs)**
  - Reference parameters
    - 5-mm/s displacement rate and 1.7-mm maximum displacement
    - Test temperatures: within 20°C to 200°C
  - Load-interrupt tests to measure permanent displacement and determine correlation between load drop and extent of cracking
    - 0.05-mm/s, stop test after significant load drop (20% to 30%)
  - Data
    - Permanent displacement: pre-test minus post-test diameter in loading direction for intact samples
    - Load-displacement curves: offset displacement, max. load, etc.
    - Normalize permanent and offset displacements to cladding outer diameter as a metric for structure “strains” (not material strain)
  - ASTM guidance document in progress

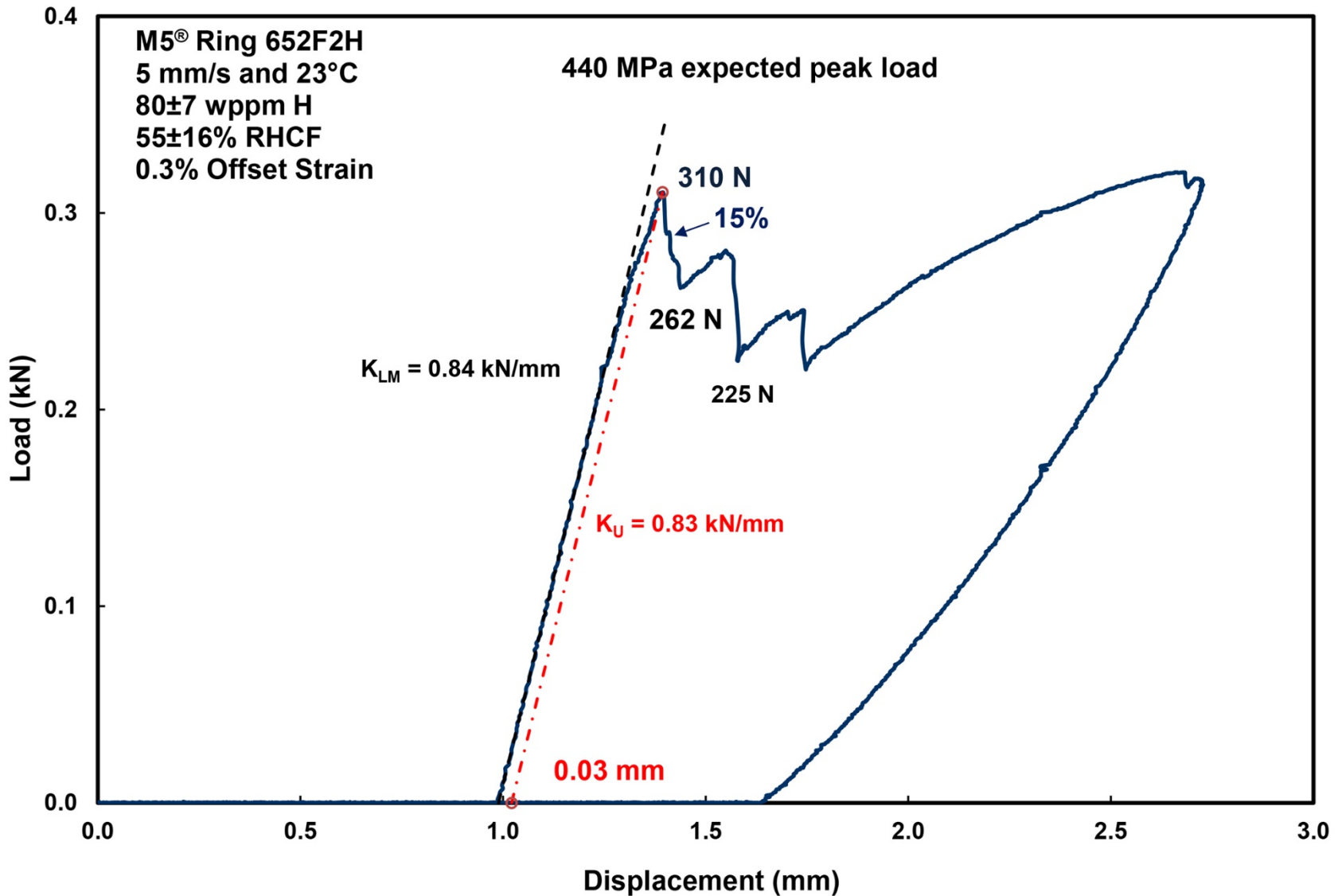
# Load-Displacement Curve for As-Fabricated M5<sup>®</sup> Benchmark Test: 25°C and 0.05 mm/s



# Load-Displacement Curve for Irradiated M5<sup>®</sup> Test after Cooling from 350°C/89-MPa: 60°C & 5 mm/s

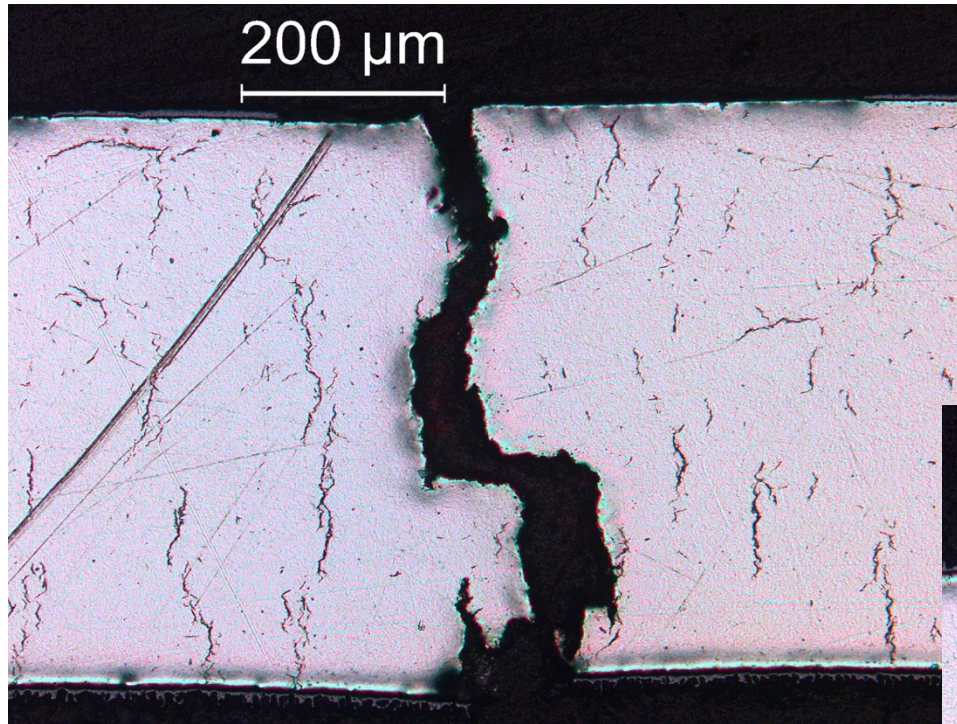


# Load-Displacement Curve for Irradiated M5<sup>®</sup> Test after Cooling from 350°C/89-MPa: 23°C & 5 mm/s



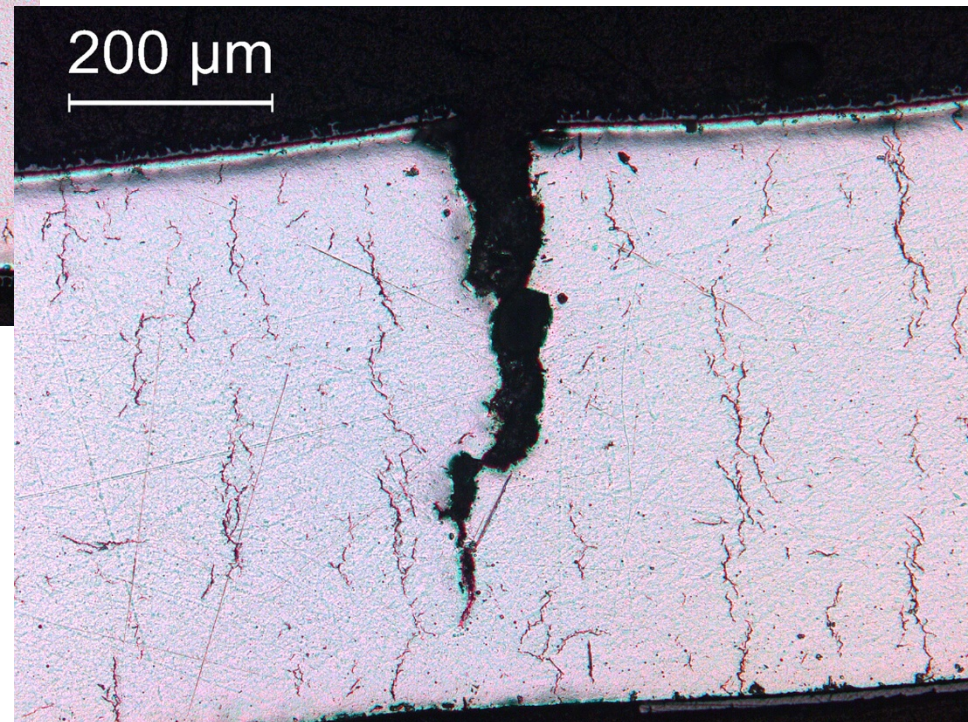


# Cracks in Irradiated M5<sup>®</sup> after Ring Compression Test at 23°C & 5 mm/s to 1.7-mm Displacement



12 o'clock position

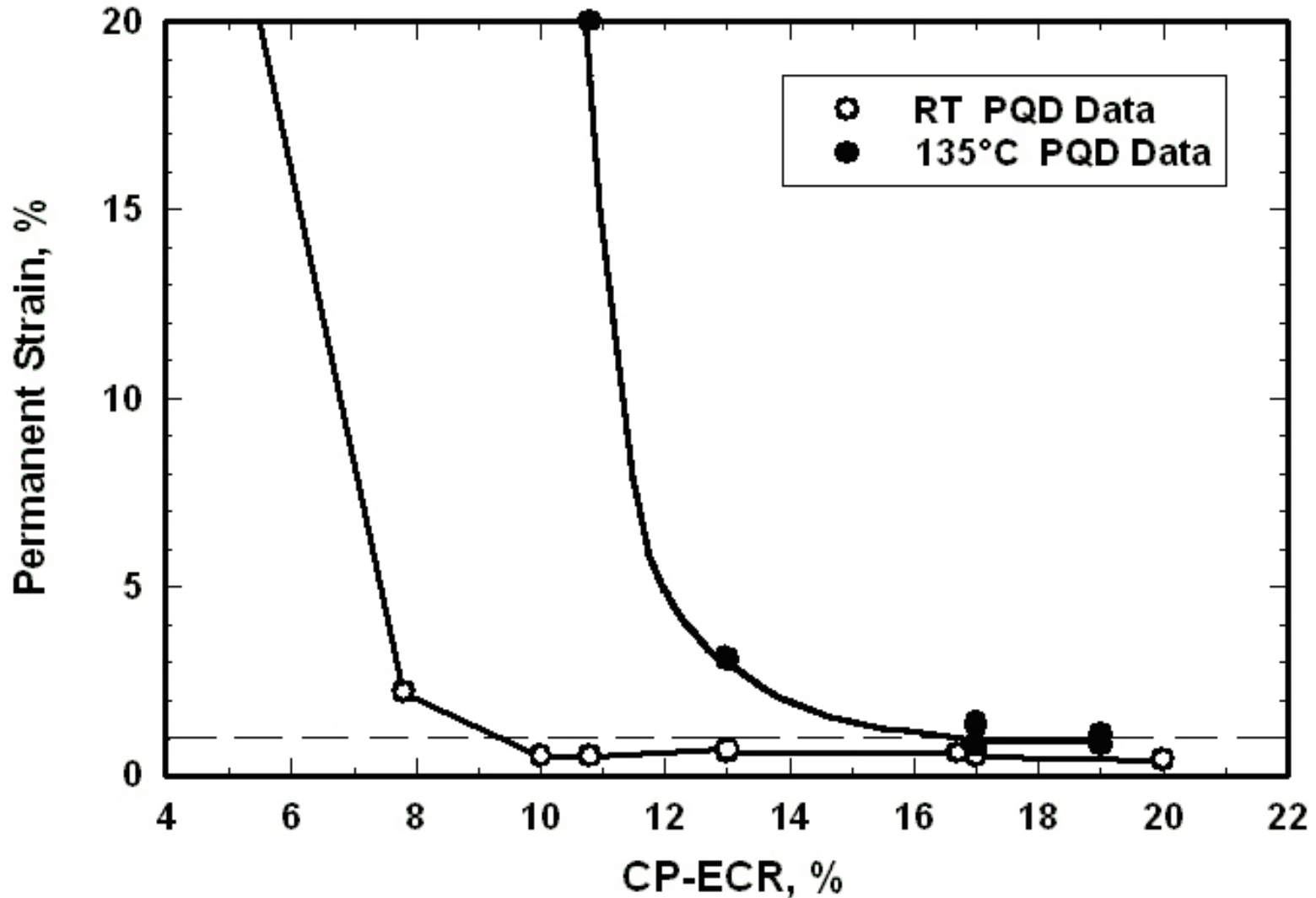
6 o'clock position



# 2% Offset Strain Ductility Criterion

- **Permanent Strain  $\geq 1\%$  Implies Ductility**
  - Observation for Loss of Coolant Accident (LOCA) studies
    - Permanent strain fluctuates in the range of  $0.5 \pm 0.4\%$  as cladding becomes more and more brittle due to increased oxygen pickup
  - Many other factors considered in criterion
- **Offset Strain  $\geq 2\%$  Implies Ductility**
  - Observation for Loss of Coolant Accident (LOCA) studies
    - Offset strain always less than permanent strain
    - Permanent strain fluctuates in the range of  $1.5 \pm 0.4\%$  as cladding becomes more and more brittle due to increased oxygen pickup
  - Many other factors considered in criterion

# Permanent Strain vs. 1200°C-Steam-Oxidation Level for 17 × 17 Zircaloy-4



# Offset Strain vs. 1200°C-Steam-Oxidation Level for 17 × 17 Zircaloy-4

